

### *Agencies and Persons Consulted*

In accordance with its stated policy, on April 19, 1995, the staff consulted with the Pennsylvania State official, Mr. Stan Maingi of the Pennsylvania Department of Environmental Resources, Bureau of Radiation Protection, regarding the environmental impact of the proposed action. The State official had no comments.

### **Finding of No Significant Impact**

Based upon the environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed amendment.

For further details with respect to this proposed action, see the licensee's letter dated September 29, 1994 and supplemental letters dated March 3, March 30, May 4 (two letters), May 8, May 9, May 16, May 24, May 25, May 26, June 7, July 7, July 13 and July 21, 1995. These letters are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW, Washington, DC 20555, and at the local public document room located at Government Publications Section, State Library of Pennsylvania, (REGIONAL DEPOSITORY) Education Building, Walnut Street and Commonwealth Avenue, Box 1601, Harrisburg, Pennsylvania 17105.

Dated at Rockville, Maryland this 8th day of August 1995.

For the Nuclear Regulatory Commission.

**John F. Stolz,**

*Director, Project Directorate I-2, Division of Reactor Projects—I/II, Office of Nuclear Reactor Regulation.*

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[Docket Nos. 50-287 and 50-388]

### **Pennsylvania Power & Light Company; Susquehanna Steam Electric Station, Units 1 and 2; Environmental Assessment and Finding of no Significant Impact**

The U.S. Nuclear Regulatory Commission (the Commission) is considering an exemption from the requirements of 10 CFR Part 50, Appendix J, Sections II.H.4, III.C.2, and III.C.3, for Facility Operating Licenses No. NPF-14 and NPF-22 respectively, issued to Pennsylvania Power & Light Company, (the licensee), for operation of the Susquehanna Steam Electric

Station (SSES), Units 1 and 2, located in Luzerne County, Pennsylvania.

### **Environmental Assessment**

#### *Identification of the Proposed Action*

The proposed action would grant an exemption from 10 CFR Part 50, Appendix J, Sections II.H.4, III.C.2, and III.C.3 for SSES, Units 1 and 2, in conjunction with the removal of the main steam isolation valve (MSIV) leakage control system (LCS) and the proposed use of an alternative pathway.

Appendix J, Sections II.H.4 and III.C.2 of 10 CFR Part 50, require leak rate testing of MSIVs at the calculated peak containment pressure related to the design basis accident, and Section III.C.3 of Appendix J requires that the measured MSIV leak rates be included in the combined local leak rate test results. The proposed deletion of the MSIV LCS and proposed use of an alternate leakage pathway affects the description of an existing exemption (NUREG-0776) which allows the leak rate testing of the MSIVs at a reduced pressure and allows the exclusion of the measured MSIV leakage from the combined local leak rate test results.

The proposed action is in accordance with the licensee's application for exemption dated February 21, 1995.

#### *The Need for the Proposed Action*

The proposed exemption is similar to the current exemption from 10 CFR Part 50, Appendix J, Sections II.H.4, III.C.2. The exemption is needed since the design of the MSIVs is such that testing in the reverse direction tends to unseat the MSIV and would result in a meaningless test. The total observed MSIV leak rate resulting from a leakage test where two MSIVs on one steam line are tested utilizing a reduced pressure (22.5 psig) will continue to be assigned to the penetration. The proposed exemption is also similar to the current exemption from 10 CFR Part 50, Appendix J, Section III.C.3. The licensee proposes that the MSIV leakage rate will continue to be accounted for separately in the radiological site analysis in accordance with the existing exemption. However, the existing exemption from 10 CFR Part 50, Appendix J, Section III.C.3 will not be applicable when the MSIV LCS is replaced with an Alternate Treatment Path (ATP) (Main steam lines and condenser).

The proposed action regarding the TS amendment will reduce the need for repairs of the MSIVs, resolve concerns associated with the current LCS performance capability at high MSIV leakage rates, and provide an effective method for dealing with a potential

MSIV leakage during a postulated loss-of-coolant accident (LOCA). Many boiling water reactors have difficulty meeting their MSIV leakage rate limits. Extensive repair, rework, and retesting efforts have negative effects on the outage costs and schedules, as well as significant impact on the licensee's as low as it is reasonable achievable (ALARA) radiological exposure programs. The alternatives proposed by the licensee to deal with the MSIV leakage make use of components (main steam lines and condenser) that are expected to remain intact and serviceable following a design basis LOCA.

#### *Environmental Impacts of the Proposed Action*

The Commission has completed its evaluation of the proposed action and concludes that this action will not increase the probability or consequences of accidents, no changes are being made in the types of any effluents that may be released offsite, and there is no significant increase in the allowable individual or cumulative occupational radiation exposure. Regarding the exemption, the MSIV leakage, along with the containment leakage is used to calculate the maximum radiological consequences of a design basis accident. Section 15.6.5 of the SSES Final Safety Analysis Report (FSAR) identifies that standard and conservative assumptions have been used to calculate the offsite and control room doses, including the doses due to MSIV leakage, which could potentially result from a postulated LOCA. Further, the control room and offsite doses resulting from a postulated LOCA have recently been recalculated using currently accepted assumptions and methods. These analyses have demonstrated that the total leakage rate of 300 scfh results in dose exposures for the control room and offsite that remain within the requirements of 10 CFR Part 100 for offsite doses and 10 CFR Part 50, Appendix A, for the control room doses.

The change will not increase the probability or consequences of accidents, no changes are being made in the types of any effluents that may be released offsite, and there is no significant increase in the allowable individual or cumulative occupational radiation exposure. Accordingly, the Commission concludes that there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential nonradiological impacts, the proposed action does involve features located entirely within the restricted area as defined in 10 CFR Part 20. It does not

affect nonradiological plant effluents and has no other environmental impact. Accordingly, the Commission concludes that there are no significant nonradiological environmental impacts associated with the proposed action.

#### *Alternatives to the Proposed Action*

Since the Commission has concluded there is no measurable environmental impact associated with the proposed action, any alternatives with equal or greater environmental impact need not be evaluated. As an alternative to the proposed action, the staff considered denial of the proposed action. Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

#### *Alternative Use of Resources*

This action does not involve the use of any resources not previously considered in the Final Environmental Statement for the Susquehanna Steam Electric Station, Units 1 and 2.

#### *Agencies and Persons Consulted*

In accordance with its stated policy, on July 7, 1995, the staff consulted with the Pennsylvania State official, David Ney of the Department of Radiation Protection, regarding the environmental impact of the proposed action. The State official had no comments.

#### **Finding of No Significant Impact**

Based upon the environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated February 21, 1995, which is available for public inspection at the Commission's Public Document Room, The Gelman Building, 2120 L Street NW., Washington, DC, and at the local public document room located at the Osterhout Free Library, Reference Department, 71 South Franklin Street, Wilkes Barre, Pennsylvania 18701.

Dated at Rockville, Maryland, this 8th day of August 1995.

For the Nuclear Regulatory Commission.

#### **John F. Stolz,**

*Director, Project Directorate I-2, Division of Reactor Projects—I/II, Office of Nuclear Reactor Regulation.*

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#### **Workshop on Development of Regulatory Guidance Implementing the Final Rule on Radiological Criteria for Decommissioning; Notice**

**SUMMARY:** The Nuclear Regulatory Commission (NRC) is preparing to hold a workshop to provide information to interested members of the public regarding draft implementation guidance for the final rule on Radiological Criteria for Decommissioning. Implementation issues to be discussed include pathways modeling, dose assessment, performance of site surveys, and instrumentation. NRC staff will discuss the use of the World Wide Web (WWW) to allow the public to interact with the staff during the development of implementation guidance. In addition, NRC staff will present an overview of the status of the final rule, including a discussion of the major issues identified in the public comments on the proposed rule (59 FR 43200).

**DATES:** The workshop will be held on Friday, September 29, 1995, from 9 a.m. until 5 p.m.

**ADDRESSES:** The workshop will be held in the main auditorium located at the NRC Headquarters Building, Two White Flint North (TWFN), 11545 Rockville Pike, Rockville, MD 20852-2738, telephone (301) 415-5811. Seating is limited to 300 people and will be on a first-come, first-served basis. It should also be noted that parking at the TWFN building is very limited.

**FOR FURTHER INFORMATION CONTACT:** Charleen Raddatz, Office of Research, Mail Stop T-9C24, U.S. Nuclear Regulatory Commission, Washington, DC 20555, telephone (301) 415-6215 or Christine Daily, Office of Research, Mail Stop T-9C24, U.S. Nuclear Regulatory Commission, Washington, DC 20555, telephone (301) 415-6026.

**SUPPLEMENTARY INFORMATION:** On August 22, 1994 (59 FR 43200), NRC published for comment a proposed rule on Radiological Criteria for Decommissioning, 10 CFR Parts 20, 30, 40, 50, 51, 70, and 72, that would provide the regulatory basis for determining the extent to which lands and structures must be remediated before a site can be considered decommissioned. Between January and June of 1993, seven workshops were conducted throughout the United States to solicit commentary from affected interested parties on the fundamental approaches and issues that must be addressed in establishing radiological criteria for decommissioning.

The purpose of this workshop is to provide information and training in

specific technical areas including pathways modeling, dose assessment, performance of site surveys and instrumentation to interested members of the public. The staff will provide additional information to facilitate the use of the World Wide Web as a means of participating in a virtual working group (VWG). The purpose of the VWG is to encourage interactive constructive input between participants and NRC staff. The VWG will be devoted to the further development of useful implementation guidance and will have the opportunity to review and comment on staff documents as they are developed. In this way, NRC staff hopes to continue the Enhanced Participatory process that has distinguished this rulemaking.

#### **Conduct of the Meeting**

The format will consist of a lecture by NRC personnel with emphasis on providing training for potential participants of the Virtual Working Group followed by time for questions and comments from the audience. Staff drafts of implementation documents will be available 30 days prior to the meeting and will be available on the NRC Enhanced Participatory Rulemaking on Radiological Criteria for Decommissioning Electronic Bulletin Board. Interested persons may connect to the bulletin board by calling 1-800-880-6091 (58 FR 37760; July 13, 1993). The bulletin board may be accessed using a personal computer, a modem, and most commonly available communications software packages. Communication software parameters should be set as follows: parity to none, data bits to 8, and stop bits to 1 (N,8,1). Use ANSI or VT-100 terminal emulation. Background documents on the rulemaking are also available for downloading and viewing on the bulletin board. For more information, call Ms. Christine Daily, U.S. Nuclear Regulatory Commission, Washington, DC 20555. Phone (301) 415-6026; FAX (301) 415-5385.

Dated at Rockville, Maryland, this 8th day of August, 1995.

For the Nuclear Regulatory Commission.

#### **John E. Glenn,**

*Chief, Radiation Protection & Health Effects Br., Division of Regulatory Applications, Office of Nuclear Regulatory Research.*

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