

regulations permit employees to remove their badges from the site, but an exemption from 10 CFR 73.55(d)(5) is required to permit contractors to take their badges offsite instead of returning them when exiting the site.

Under the proposed system, all individuals authorized to gain unescorted access will have the physical characteristics of their hand (hand geometry) recorded with their badge number. Since the hand geometry is unique to each individual and its application in the entry screening function would preclude unauthorized use of a badge, the requested exemption would allow employees and contractors to keep their badges at the time of exiting the protected area. The process of verifying badge issuance, ensuring badge retrieval, and maintaining badges could be eliminated while the balance of the access procedure would remain intact. Firearm, explosive, and metal detection equipment and provisions for conducting searches will remain as well. The security officer responsible for the last access control function (controlling admission to the protected area) will also remain isolated within a bullet-resistant structure in order to assure his or her ability to respond or to summon assistance.

Use of a hand geometry biometrics system exceeds the present verification methodology's capability to discern an individual's identity. Unlike the photograph identification badge, hand geometry is nontransferable. During the initial access authorization or registration process, hand measurements are recorded and the template is stored for subsequent use in the identity verification process required for entry into the protected area. Authorized individuals insert their access authorization card into the card reader and the biometrics system records an image of the hand geometry. The unique features of the newly recorded image are then compared to the template previously stored in the database. Access is ultimately granted based on the degree to which the characteristics of the image match those of the "signature" template.

Since both the badge and hand geometry would be necessary for access into the protected area, the proposed system would provide for a positive verification process. Potential loss of a badge by an individual, as a result of taking the badge offsite, would not enable an unauthorized entry into protected areas.

The access process will continue to be under the observation of security personnel. The system of identification badges coupled with their associated

access control cards will continue to be used for all individuals who are authorized access to protected areas without escorts. Badges will continue to be displayed by all individuals while inside the protected area. Addition of a hand geometry biometrics system will provide a significant contribution to effective implementation of the security plan at each site.

#### IV

For the foregoing reasons, pursuant to 10 CFR 73.55, the NRC staff has determined that the proposed alternative measures for protection against radiological sabotage meet "the same high assurance objective," and "the general performance requirements" of the regulation and that "the overall level of system performance provides protection against radiological sabotage equivalent" to that which would be provided by the regulation.

Accordingly, the Commission has determined that, pursuant to 10 CFR 73.5, an exemption is authorized by law, will not endanger life or property or common defense and security, and is otherwise in the public interest. Therefore, as long as the licensee uses the hand geometry access control system, the Commission hereby grants Entergy Operations, Inc. an exemption from those requirements of 10 CFR 73.55(d)(5) relating to the returning of picture badges upon exit from the protected area such that individuals not employed by the licensee, i.e., contractors, who are authorized unescorted access into the protected area, can take their badges offsite.

Pursuant to 10 CFR 51.32, the Commission has determined that the granting of this exemption will have no significant impact on the quality of the human environment (60 FR 40865). This exemption is effective upon issuance.

Dated at Rockville, Maryland, this 11th day of August 1995.

For The Nuclear Regulatory Commission.

**Elinor G. Adensam,**

*Acting Director, Division of Reactor Projects III/IV, Office of Nuclear Reactor Regulation.*

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[Docket No. 50-289]

#### **GPU Nuclear Corporation; Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing**

The U.S. Nuclear Regulatory Commission (the Commission) is

considering issuance of an amendment to Facility Operating License No. DPR-50 issued to GPU Nuclear Corporation (the licensee) for operation of the Three Mile Island Nuclear Station, Unit 1 (TMI-1) located in Dauphin County, Pennsylvania.

The proposed amendment would remove Technical Specification (TS) Section 3.2, "Makeup and Purification and Chemical Addition Systems," and its bases. The pertinent requirements and bases applicable to these systems are being incorporated in the TMI-1 Updated Final Safety Analysis Report (UFSAR). This proposed change is consistent with the Standard Technical Specifications for Babcock and Wilcox Plants (NUREG-1430, September 1992), which do not include requirements for these systems. The proposed change is also consistent with the Commission's criteria to be used to determine which structures, systems, and components are to be included in the TS. These criteria were recently codified in 10 CFR 50.36 of the Commission's regulations as noticed in the **Federal Register** (60 FR 36953, July 19, 1995). The licensee's request for the amendment under consideration is dated August 11, 1995, and supersedes an earlier request dated May 17, 1995. The staff had noticed the earlier request in the **Federal Register** on June 21, 1995 (60 FR 32365).

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Operation of the facility in accordance with the proposed amendment would not involve a significant increase in the probability of occurrence or consequences of an accident previously evaluated. The administrative relocation of the existing Technical Specification 3.2 requirements for the Makeup and Purification and Chemical Addition Systems to the TMI-1 UFSAR is

unrelated to the probability of occurrence or the consequences of an accident previously evaluated. Design basis accident and transient analysis criteria regarding emergency core cooling system (ECCS) cold shutdown boration requirements are maintained in TMI-1 Technical Specification Section 3.3. The requirements currently contained in Technical Specification 3.2 do not meet the criteria in the NRC "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors," July 1992, as codified by the revision to 10 CFR 50.36. The proposed amendment is expected to produce an improvement in safety through reduced potential action statement induced plant transients. Therefore, the proposed amendment has no effect on the probability of occurrence or consequences of an accident previously evaluated.

2. Operation of the facility in accordance with the proposed amendment would not create the possibility of a new or different kind of accident from any accident previously evaluated. Design basis accident and transient analysis criteria regarding ECCS cold shutdown boration requirements are maintained in TMI-1 Technical Specification Section 3.3. Administrative relocation of the existing Technical Specification 3.2 requirements for the Makeup and Purification and Chemical Addition Systems to the UFSAR ensures that these system requirements and bases are appropriately controlled in accordance with the requirements of 10 CFR 50.59. Therefore, the proposed amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. Operation of the facility in accordance with the proposed amendment would not involve a significant reduction in a margin of safety. The proposed amendment is consistent with the Standard Technical Specifications for Babcock and Wilcox Plants, NUREG-1430, July 1992, and the NRC Final Policy Statement on Improvements to Technical Specifications. The requirements currently contained in TMI-1 Technical Specification Section 3.2 do not meet any of the four (4) criteria in the Final Policy Statement for inclusion in Technical Specifications, as codified in the revision to 10 CFR 50.36. The proposed amendment is expected to produce an improvement in safety through reduced potential action statement induced plant transients. Administrative relocation of the existing TMI-1 Technical Specification Section 3.2 requirements for the Makeup and Purification and Chemical Addition Systems to the UFSAR ensures that these system requirements and bases are appropriately controlled in accordance with the requirements of 10 CFR 50.59. Therefore, it is concluded that operation of the facility in accordance with the proposed amendment does not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are

satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the **Federal Register** a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Rules Review and Directives Branch, Division of Freedom of Information and Publications Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and should cite the publication date and page number of this **Federal Register** notice. Written comments may also be delivered to Room 6D22, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street NW., Washington, DC.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By September 18, 1995, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10

CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Law/Government Publications Section, State Library of Pennsylvania, (Regional Depository) Walnut Street and Commonwealth Avenue, Box 1601, Harrisburg, PA 17105.

If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) The nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to

rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. Where petitions are filed during the last 10 days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-(800) 248-5100 (in Missouri 1-(800) 342-6700). The Western Union operator should be given Datagram Identification Number N1023 and the following message addressed to Phillip F. McKee: petitioner's name and telephone number, date petition was mailed, plant name, and publication

date and page number of this **Federal Register** notice. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and to Ernest L. Blake, Jr., Esquire, Shaw, Pittman, Potts & Trowbridge, 2300 N Street, NW., Washington, DC 20037, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1) (i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated August 11, 1995, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Law/Government Publications Section, State Library of Pennsylvania, (Regional Depository) Walnut Street and Commonwealth Avenue, Box 1601, Harrisburg, PA 17105.

Dated at Rockville, Maryland, this day of August 1995.

For The Nuclear Regulatory Commission.

**Ronald W. Hernan,**

*Senior Project Manager, Project Directorate I-3, Division of Reactor Projects—I/II, Office of Nuclear Reactor Regulation.*

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### **Proposed Generic Letter; Revised Contents of the Monthly Operating Report**

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Notice of opportunity for public comment.

**SUMMARY:** The Nuclear Regulatory Commission (NRC) is proposing to issue a generic letter regarding revised monthly operating report (MOR) contents. The purpose of the proposed generic letter is to inform licensees of nuclear power reactors that the NRC is requesting the submittal, on a voluntary basis, of less information in a modified version of the monthly operating report.

In a staff requirements memorandum (SRM) dated April 26, 1994, in which the Commission endorsed the NRC staff proposal in SECY-94-093 to assess the reporting requirements for power

reactor licensees and initiate rulemaking or other appropriate actions consistent with the plan for implementing the recommendations of the Regulatory Review Group (SECY-94-003), the Commission cautioned NRC staff to "consider the public's need for the information in assessing the body of reporting requirements." Therefore, the NRC is seeking comment from interested parties on the generic letter presented under the Supplementary Information heading, regarding the need to (1) retain information deleted from the monthly operating report identified in Draft Regulatory Guide 1.16/Revision 4, "Reporting of Operating Information-Appendix A Technical Specifications," or (2) further restrict or more explicitly define the information to be reported in the monthly operating report. As an example, in focusing on the reporting of performance indicator data in the MOR, information needed to calculate unit availability will no longer be reported; this includes the unit reserve shutdown hours and the hours in the reporting period. While this statistic may be of interest to certain elements of the industry or public, it is not an essential part of the safety mission of the agency to continue to compile this information.

This generic letter was endorsed by the Committee to Review Generic Requirements (CRGR) on August 1, 1995. The relevant information that was sent to the CRGR will be placed in the Public Document Room. The NRC will consider comments received from interested parties in the final evaluation of the proposed generic letter. The final evaluation by the NRC will include a review of the technical position and, as appropriate, an analysis of the value/impact on licensees. Should this generic letter be issued by the NRC, it will become available for public inspection in the Public Document Room.

**DATES:** Comment period expires on September 18, 1995. Comments submitted after this date will be considered if it is practical to do so; assurance of consideration can only be given for those comments received on or before this date.

**ADDRESSES:** Submit written comments to Chief, Rules Review and Directives Branch, U.S. Nuclear Regulatory Commission, Washington, DC 20555. Written comments may also be delivered to 11545 Rockville Pike, Rockville, Maryland, from 7:30 am to 4:15 pm, Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, 2120 L Street, NW, (Lower Level), Washington, DC.