

Brief description of amendment: The amendment revises the Technical Specifications to exclude the requirement to perform the slave relay test of the 36-inch containment purge supply and exhaust valves on a quarterly basis while the plant is in Modes 1, 2, 3, or 4.

Date of issuance: September 18, 1995

Effective date: September 18, 1995

Amendment No.: 128

Facility Operating License No. NPF-12. Amendment revises the Technical Specifications.

Date of initial notice in Federal Register: August 16, 1995 (60 FR 42608) The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 18, 1995. No significant hazards consideration comments received: No

Local Public Document Room location: Fairfield County Library, 300 Washington Street, Winnsboro, SC 29180

South Carolina Electric & Gas Company, South Carolina Public Service Authority, Docket No. 50-395, Virgil C. Summer Nuclear Station, Unit No. 1, Fairfield County, South Carolina

Date of application for amendment: June 19, 1995, as supplemented on August 21, 1995.

Brief description of amendment: The amendment revises the Technical Specifications to change the required test frequency for the reactor building spray nozzle flow test from once per five years to once per ten years.

Date of issuance: September 18, 1995

Effective date: September 18, 1995

Amendment No.: 129

Facility Operating License No. NPF-12. Amendment revises the Technical Specifications.

Date of initial notice in Federal Register: July 19, 1995 (60 FR 37100). The August 21, 1995 letter provided supplemental information that did not change the initial proposed no significant hazards consideration. The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 18, 1995. No significant hazards consideration comments received: No

Local Public Document Room location: Fairfield County Library, 300 Washington Street, Winnsboro, SC 29180

The Cleveland Electric Illuminating Company, Centerior Service Company, Duquesne Light Company, Ohio Edison Company, Pennsylvania Power Company, Toledo Edison Company, Docket No. 50-440, Perry Nuclear Power Plant, Unit No. 1, Lake County, Ohio

Date of application for amendment: April 3, 1995

Brief description of amendment: The amendment revised the Technical Specifications (TS) to relocate radiological effluent and radiological environmental monitoring TS to the Offsite Dose Calculation Manual or to the Process Control Program.

Programmatic controls for radioactive effluent and radiological environmental monitoring were included in TS 6.8.4.

Date of issuance: September 15, 1995

Effective date: September 15, 1995

Amendment No.: 72

Facility Operating License No. NPF-58: This amendment revised the Technical Specifications.

Date of initial notice in Federal Register: May 10, 1995 (60 FR 24921) The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 15, 1995. No significant hazards consideration comments received: No

Local Public Document Room location: Perry Public Library, 3753 Main Street, Perry, Ohio 44081

The Cleveland Electric Illuminating Company, Centerior Service Company, Duquesne Light Company, Ohio Edison Company, Pennsylvania Power Company, Toledo Edison Company, Docket No. 50-440, Perry Nuclear Power Plant, Unit No. 1, Lake County, Ohio

Date of application for amendment: June 1, 1995

Brief description of amendment: The amendment revised the Technical Specifications to make them more restrictive regarding control rod drive scram time testing. CRD scram time testing would be required following maintenance prior to considering the CRD operable, and could be performed at any reactor pressure. Additional testing would be required when reactor coolant pressure is greater than or equal to 950 psig and prior to 40 percent rated thermal power.

Date of issuance: September 26, 1995

Effective date: September 26, 1995

Amendment No.: 73

Facility Operating License No. NPF-58: This amendment revised the Technical Specifications.

Date of initial notice in Federal Register: August 2, 1995 (60 FR 39452)

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 26, 1995. No significant hazards consideration comments received: No

Local Public Document Room location: Perry Public Library, 3753 Main Street, Perry, Ohio 44081

Washington Public Power Supply System, Docket No. 50-397, Nuclear Project No. 2, Benton County, Washington

Date of application for amendment: January 14, 1992, as supplemented by letters dated February 10, 1995, and August 16, 1995.

Brief description of amendment: The amendment revises technical specification surveillance requirements regarding demonstration of jet pump operability and corrects several administrative discrepancies.

Date of issuance: September 18, 1995

Effective date: September 18, 1995, to be implemented within 30 days of issuance

Amendment No.: 141

Facility Operating License No. NPF-21: The amendment revised the Technical Specifications.

Date of initial notice in Federal Register: May 27, 1992 (57 FR 22272) and March 29, 1995 (60 FR 16204). The August 16, 1995, supplemental letter provided additional clarifying information and did not change the initial no significant hazards consideration determination. The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 18, 1995. No significant hazards consideration comments received: No

Local Public Document Room location: Richland Public Library, 955 Northgate Street, Richland, Washington 99352

Dated at Rockville, Maryland, this 3rd day of October 1995.

For the Nuclear Regulatory Commission
Elinor G. Adensam,
Deputy Director, Division of Reactor Projects - III/IV, Office of Nuclear Reactor Regulation
[Doc. 95-25006 Filed 10-10-95; 8:45 am]

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[Docket No. 50-251]

Florida Power and Light Company (Turkey Point Unit 4); Exemption

I

Florida Power and Light Company (the licensee) is the holder of Facility Operating License No. DPR-41, which authorizes operation of Turkey Point Unit 4 (the facility), at a steady-state

reactor power level not in excess of 2200 megawatts thermal. The facility is a pressurized water reactor located at the licensee's site in Dade County, Florida. The license provides among other things, that it is subject to all rules, regulations, and Orders of the U.S. Nuclear Regulatory Commission (the Commission or NRC) now or hereafter in effect.

II

Section III.D.1.(a) of Appendix J to 10 CFR Part 50 requires the performance of three Type A containment integrated leakage rate tests (ILRTs) of the primary containment, at approximately equal intervals during each 10-year service period.

III

By letter dated August 8, 1995, and revised by letter dated September 6, 1995, the licensee requested an exemption from the requirements pertaining to the Type A testing interval required by 10 CFR 50 Appendix J. This section requires the performance of three Type A tests of the primary containment at approximately equal intervals during each 10-year service period. The requested exemption would permit a one-time interval extension of the Type A test by one refueling outage (from the March 1996 refueling outage, to the October 1997 refueling outage).

The licensee's request cites the special circumstances of 10 CFR 50.12, paragraph (a)(2)(ii) as the basis for the exemption. The licensee points out that the existing Type B and C testing programs are not being modified by this request and allowing a one-time scheduler exemption will not reduce the current level of safety since the Type A test frequency does not alter the containment leak rates.

IV

In the licensee's August 8, 1995, as revised by letter dated September 6, 1995, exemption request, the licensee stated that special circumstance 50.12(a)(2)(ii) is applicable to this situation, i.e., that application of the regulation is not necessary to achieve the underlying purpose of the rule.

Appendix J states that the leakage test requirements provide for periodic verification by tests of the leak tight integrity of the primary reactor containment. Appendix J further states that the purpose of the tests "is to assure that leakage through the primary reactor containment shall not exceed the allowable leakage rate values as specified in the Technical Specifications or associated bases." Thus, the underlying purpose of the

requirement to perform Type A containment leak rate tests at intervals during the 10-year service period is to ensure that any potential leakage pathways through the containment boundary are identified within a time span that prevents significant degradation from continuing or becoming unknown.

The NRC staff has reviewed the basis and supporting information provided by the licensee in the exemption request. It has been the experience at Turkey Point Unit 4 during the Type A tests conducted from 1982 to date, that the Type A tests have demonstrated that the reactor containment buildings have acceptable leak rates that are far below the leak rates assumed in the site's offsite dose calculation and the ILRT acceptance criteria. The licensee has reported that the test results are approximately one-third to one-fourth of the leakage assumed in offsite dose rate calculations (0.25%) and approximately one-half to one-third of the acceptance criteria for the ILRT (0.1875%). The leak rate data from these tests do not show an increasing trend, indicating that the containment liner and isolation system are stable and supporting the conclusion that a one-time scheduler exemption will not reduce the current level of safety.

The licensee will perform the general containment inspection although it is only required by Appendix J (Section V.A.) to be performed in conjunction with Type A tests. The NRC staff considers that these inspections, though limited in scope, provide an important added level of confidence in the continued integrity of the containment boundary.

The NRC staff has also made use of a draft staff report, NUREG-1493, which provides the technical justification for the present Appendix J rulemaking effort which also includes a 10-year test interval for Type A tests. The integrated leakage rate test, or Type A test, measures overall containment leakage. However, operating experience with all types of containments used in this country demonstrates that essentially all containment leakage can be detected by local leakage rate tests (Type B and C). According to results given in NUREG-1493, out of 180 ILRT reports covering 110 individual reactors and approximately 770 years of operating history, only 5 ILRT failures were found which local leakage rate testing could not detect. This is 3% of all failures. This study agrees well with previous NRC staff studies which show that Type B and C testing can detect a very large percentage of containment leaks.

The Nuclear Management and Resources Council (NUMARC), now the Nuclear Energy Institute (NEI), collected and provided the NRC staff with summaries of data to assist in the Appendix J rulemaking effort. NUMARC collected results of 144 ILRTs from 33 units; 23 ILRTs exceeded $1.0L_a$. Of these, only nine were not due to Type B or C leakage penalties. The NEI data also added another perspective. The NEI data show that in about one-third of the cases exceeding allowable leakage, the as-found leakage was less than $2L_a$; in one case the leakage was found to be approximately $2L_a$; in one case the as-found leakage was less than $3L_a$; one case approached $10L_a$; and in one case the leakage was found to be approximately $21L_a$. For about half of the failed ILRTs the as-found leakage was not quantified. These data show that, for those ILRTs for which the leakage was quantified, the leakage values are small in comparison to the leakage value at which the risk to the public starts to increase over the value of risk corresponding to L_a (approximately $200L_a$, as discussed in NUREG-1493). Therefore, based on those considerations, it is unlikely that an extension of one cycle for the performance of the Appendix J, Type A test at Turkey Point Unit 4 would result in significant degradation of the overall containment integrity. As a result, the application of the regulation in these particular circumstances is not needed to achieve the underlying purpose of the rule.

Based on generic and plant-specific data, the NRC staff finds the basis for the licensee's proposed exemption to allow a one-time exemption to permit a scheduler extension of one cycle for the performance of the Appendix J Type A test, provided that the general containment inspection is performed, to be acceptable.

Pursuant to 10 CFR 51.32, the Commission has determined that granting this Exemption will not have a significant impact on the environment (60 FR 49926).

This Exemption is effective upon issuance and shall expire at the completion of the 1997 refueling outage.

Dated at Rockville, Maryland, this 27th day of September 1995.

For the Nuclear Regulatory Commission.

Steven A. Varga,

*Director, Division of Reactor Projects—I/II,
Office of Nuclear Reactor Regulation.*

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