

The agenda for the subject meeting shall be as follows:

Tuesday, October 31, 1995—8:30 a.m. until the conclusion of business.

The Subcommittee will continue its review of the emergency procedure guidelines developed by the BWR Owners Group (BWROG) to mitigate an ATWS event compounded by core power instability. The purpose of this meeting is to gather information, analyze relevant issues and facts, and to formulate proposed positions and actions, as appropriate, for deliberation by the full Committee.

Oral statements may be presented by members of the public with the concurrence of the Subcommittee Chairman; written statements will be accepted and made available to the Committee. Electronic recordings will be permitted only during those portions of the meeting that are open to the public, and questions may be asked only by members of the Subcommittee, its consultants, and staff. Persons desiring to make oral statements should notify the cognizant ACRS staff engineer named below five days prior to the meeting, if possible, so that appropriate arrangements can be made.

During the initial portion of the meeting, the Subcommittee, along with any of its consultants who may be present, may exchange preliminary views regarding matters to be considered during the balance of the meeting.

The Subcommittee will then hear presentations by and hold discussions with representatives of the NRC staff, BWROG, General Electric Nuclear Energy, their consultants, and other interested persons regarding this review.

Further information regarding topics to be discussed, whether the meeting has been cancelled or rescheduled, the scheduling of sessions which are open to the public, the Chairman's ruling on requests for the opportunity to present oral statements and the time allotted therefor can be obtained by contacting the cognizant ACRS staff engineer, Mr. Paul A. Boehnert (telephone 301/415-8065) between 7:30 a.m. and 4:15 p.m. EDT). Persons planning to attend this meeting are urged to contact the above named individual one or two working days prior to the meeting to be advised of any potential changes in the proposed agenda, etc., that may have occurred.

Dated: October 12, 1995.

Sam Duraiswamy,

Chief, Nuclear Reactors Branch.

[FR Doc. 95-25802 Filed 10-17-95; 8:45 am]

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Correction to Biweekly Notice Applications and Amendments to Facility Operating Licenses Involving No Significant Hazards Considerations

On October 11, 1995, the Federal Register published the Biweekly Notice of Applications and Amendments to Facility Operating Licenses Involving No Significant Hazards Considerations. On page 52927, Column 2, Paragraph 2, the first line should read as follows: "By November 13, 1995, the licensee."

Dated at Rockville, Maryland, this 12th day of October 1995.

For the Nuclear Regulatory Commission,
Elinor G. Adensam,

Deputy Director Division of Reactor Projects—III/IV, Office of Nuclear Reactor Regulation.

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[Docket No. 50-289]

GPU Nuclear Corporation, et al.; Three Mile Island Nuclear Station, Unit No. 1; Exemption

I

GPU Nuclear Corporation (the licensee) is the holder of Facility Operating License No. DPR-50, which authorizes operation of Three Mile Island Nuclear Station, Unit No. 1 (TMI-1). The license provides, among other things, that the licensee be subject to all rules, regulations, and Orders of the Nuclear Regulatory Commission (the Commission) now or hereafter in effect.

The facility consists of a pressurized water reactor at the licensee's site located in Dauphin County, Pennsylvania.

II

By letter dated June 1, 1995, the licensee requested an exemption to 10 CFR 50.44, 10 CFR 50.46, and Appendix K to 10 CFR Part 50 that would enable the use of two demonstration assemblies during TMI-1 Cycles 11, 12, and 13. These regulations refer to pressurized water reactors fueled with uranium oxide pellets within cylindrical zircaloy or ZIRLO cladding. The two demonstration assemblies to be used during these fuel cycles contain fuel rods with zirconium-based claddings that are not chemically identical to zircaloy or ZIRLO.

Since 10 CFR 50.46 and Appendix K to 10 CFR Part 50 identify requirements for calculating emergency core cooling system (ECCS) performance for reactors containing fuel with zircaloy or ZIRLO cladding, and 10 CFR 50.44 relates to the generation of hydrogen gas from a metal-water reactor with reactor fuel

having zircaloy or ZIRLO cladding, an exemption is required to place the two demonstration assemblies containing fuel rods with advanced zirconium based cladding in the core.

III

Title 10 of the Code of Federal Regulations at 50.12(a)(2)(ii) enables the Commission to grant an exemption from the requirements of Part 50 when special circumstances are present such that application of the regulation in the particular circumstances would not serve the underlying purpose of the rule, or is not necessary to achieve the underlying purpose of the rule. The underlying purpose of 10 CFR 50.46 and 10 CFR Part 50, Appendix K is to establish requirements for the calculation of ECCS performance. The licensee has performed a calculation demonstrating adequate ECCS performance for TMI-1 and has shown that the two demonstration assemblies do not have a significant impact on that previous calculation. As such, the licensee has achieved the underlying purpose of 10 CFR 50.46 and 10 CFR Part 50, Appendix K. The underlying purpose of 10 CFR 50.44 is to ensure that means are provided for the control of hydrogen gas that may be generated following a postulated loss-of-coolant accident. The licensee has provided means for controlling hydrogen gas and has previously considered the potential for hydrogen gas generation stemming from a metal-water reaction. The small number of fuel rods in the two demonstration assemblies containing advanced zirconium-based claddings in conjunction with the chemical similarity of the advanced claddings to zircaloy and ZIRLO ensures that previous calculations of hydrogen production resulting from a metal-water reactor would not be significantly changed. As such, the licensee has achieved the underlying purpose of 10 CFR 50.44.

The two demonstration assemblies that will be placed in the TMI-1 reactor during Cycles 11, 12, and 13 meet the same design bases as the fuel in the reactor during previous cycles. No safety limits or setpoints have been altered as result of the use of the two demonstration assemblies. The demonstration assemblies will be placed in core locations that will not experience limiting power peaking during Cycles 11, 12, or 13. The advanced claddings have been tested for corrosion resistance, tensile and burst strength, and creep characteristics. The results indicate that the advanced claddings are safe for reactor service.