

2. Current OMB approval number:
3150-0056
3. How often the collection is required:
Once.
4. Who is required or asked to report:
NRC licensed or certified facilities on the U. S. eligible list who have been notified in writing by the Commission to submit the form.
5. The number of annual respondents:
One.
6. The number of hours needed annually to complete the requirement or request: 360.
7. Abstract: NRC licensed or certified facilities that appear on the U. S. eligible list, pursuant to the US/IAEA Safeguards Agreement, and who have been notified in writing by the Commission, are required to complete and submit a Design Information Questionnaire, IAEA Form N-71, to provide information concerning their installation for use of the International Atomic Energy Agency. Submit, by January 23, 1996, comments that address the following questions:

1. Is the proposed collection of information necessary for the NRC to properly perform its functions? Does the information have practical utility?
2. Is the burden estimate accurate?
3. Is there a way to enhance the quality, utility, and clarity of the information to be collected?
4. How can the burden of the information collection be minimized, including the use of automated collection techniques or other forms of information technology?

A copy of the draft supporting statement may be viewed free of charge at the NRC Public Document Room, 2120 L Street NW, (lower level), Washington, DC. Members of the public who are in the Washington, DC, area can access this document via modem on the Public Document Room Bulletin Board (NRC's Advance Copy Document Library), NRC subsystem, at FedWorld, 703-321-3339. Members of the public who are located outside of the Washington, DC, area can dial FedWorld, 1-800-303-9672, or use the FedWorld Internet address: fedworld.gov (Telnet). The document will be available on the bulletin board for 30 days after the signature date of this notice. If assistance is needed in accessing the document, please contact the FedWorld help desk at 703-487-4608.

Comments and questions may be directed to the NRC Clearance Officer, Brenda Jo. Shelton, U.S. Nuclear Regulatory Commission, T-6 F33, Washington, DC, 20555-0001, or by telephone at (301) 415-7233, or by Internet electronic mail at bjs1@nrc.gov.

Dated at Rockville, Maryland, this 14th day of November 1995.

For the Nuclear Regulatory Commission.
Gerald F. Cranford,
Designated Senior Official for Information Resources Management.
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[Docket Nos. 50-003 and 50-247]

Consolidated Edison Company of New York (Indian Point Nuclear Generating Unit Nos. 1 and 2); Exemption

I

Consolidated Edison Company of New York, Inc., (the licensee) is the holder of Facility Operating License No. DPR-26 which authorizes the operation of Indian Point Nuclear Generating Unit No. 2 and Provisional Operating License (POL) No. DPR-5 which authorizes the operation of Indian Point Nuclear Generating Unit No. 1. The operating authority of POL DPR-5 for Indian Point Nuclear Generating Unit No. 1 was revoked by Commission Order dated June 19, 1980. The operating licenses provide, among other things, that the licensee is subject to all rules, regulations, and orders of the Commission now and hereafter in effect.

The facility comprises two pressurized-water reactors at the licensee's site in Westchester County, New York.

II

The Code of Federal Regulations at 10 CFR 73.55, "Requirements for physical protection of licensed activities in nuclear power reactors against radiological sabotage," paragraph (a), in part, states that "The licensee shall establish and maintain an onsite physical protection system and security organization which will have as its objective to provide high assurance that activities involving special nuclear material are not inimical to the common defense and security and do not constitute an unreasonable risk to the public health and safety."

Paragraph (1), "Access Requirements," of 10 CFR 73.55(d), specifies that "The licensee shall control all points of personnel and vehicle access into a protected area." 10 CFR 73.55(d)(5) requires that "A numbered picture badge identification system shall be used for all individuals who are authorized access to protected areas without escort." 10 CFR 73.55(d)(5) also states that an individual not employed by the licensee (i.e., contractors) may be authorized access to protected areas without escort provided

the individual "receives a picture badge upon entrance into the protected area which must be returned upon exit from the protected area * * *"

The licensee proposed to implement an alternative unescorted access control system which would eliminate the need to issue and retrieve badges at the entrance into the protected area and would allow all individuals with unescorted access to keep their badge with them when departing the site.

An exemption from 10 CFR 73.55(d)(5) is required to allow contractors who have unescorted access to take their badges offsite instead of returning them when exiting the site. By letter dated August 10, 1995, the licensee requested an exemption from certain requirements of 10 CFR 73.55(d)(5) for this purpose.

III

Pursuant to 10 CFR 73.5, "Specific exemptions," the Commission may, upon application of any interested person or upon its own initiative, grant such exemptions from the requirements of the regulations in this part as it determines are authorized by law and will not endanger life or property or the common defense and security, and are otherwise in the public interest.

Pursuant to 10 CFR 73.55, the Commission may authorize a licensee to provide alternative measures for protection against radiological sabotage provided the licensee demonstrates that the alternative measures have "the same high assurance objective" and meet "the general performance requirements" of the regulation, and "the overall level of system performance provides protection against radiological sabotage equivalent" to that which would be provided by the regulation.

Currently, employee and contractor identification badges/keycards, are issued and retrieved on the occasion of each entry to and exit from the protected areas of the Indian Point Nuclear Generating Unit Nos. 1 and 2 site. Station security personnel are required to maintain control of the badges/keycards while the individuals are offsite. This practice has been in effect at Indian Point Nuclear Generating Unit Nos. 1 and 2 since the operating licenses were issued. Security personnel retain each identification badge/keycard when not in use by the authorized individual, within appropriately designed storage receptacles. An individual who meets the access authorization requirements is issued an individual picture badge/keycard which allows entry into preauthorized areas of the station. While entering the plant in the present

configuration, an authorized individual is "screened" by the required detection equipment and by the issuing security officer. Having received the picture badge/keycard, the individual proceeds to the access portal, inserts the picture badge/keycard into the card reader, and passes through the turnstile which unlocks if the preset criteria are met.

This present procedure is labor intensive since security personnel are required to verify badges/keycards issuance, ensure badges/keycards retrieval, and maintain the badges/keycards in orderly storage until the next entry into the protected area. The regulations permit employees to remove their badges/keycards from the site, but an exemption from 10 CFR 73.55(d)(5) is required to permit contractors to take their badges/keycards offsite instead of returning them when exiting the site.

Under the proposed system, all individuals authorized to gain unescorted access will have the physical characteristics of their hand (hand geometry) recorded with their badge/keycard. Since the hand geometry is unique to each individual and its application in the entry screening function would preclude unauthorized use of a badge/keycard, the requested exemption would allow employees and contractors to keep their badges/keycards at the time of exiting the protected area. The process of verifying badge/keycard issuance, ensuring badge/keycard retrieval, and maintaining badges/keycards could be eliminated while the balance of the access procedure would remain intact. Firearm, explosive, and metal detection equipment and provisions for conducting searches will remain as well. The security officer responsible for the last access control function (controlling admission to the protected area) will also remain isolated within a bullet-resistant structure in order to assure his or her ability to respond or to summon assistance.

Use of a hand geometry biometrics system exceeds the present verification methodology's capability to discern an individual's identity. Unlike the photograph identification badge/keycard, hand geometry is nontransferable. During the initial access authorization or registration process, hand measurements are recorded and the template is stored for subsequent use in the identity verification process required for entry into the protected area.

Authorized individuals insert their picture badges/keycards into the card reader and the biometrics system records an image of the hand geometry. The unique features of the newly

recorded image are then compared to the template previously stored in the database. Access is ultimately granted based on the degree to which the characteristics of the image match those of the "signature" template.

Since both the badges/keycards and hand geometry would be necessary for access into the protected area, the proposed system would provide for a positive verification process. Potential loss of a badge/keycard by an individual, as a result of taking the badge/keycard offsite, would not enable an unauthorized entry into protected areas.

The access process will continue to be under the observation of security personnel. The system of identification badges/keycards will continue to be used for all individuals who are authorized access to protected areas without escorts. Badges/keycards will continue to be displayed by all individuals while inside the protected area. Addition of a hand geometry biometrics system will provide a significant contribution to effective implementation of the security plan at the site.

IV

For the foregoing reasons, pursuant to 10 CFR 73.55, the NRC staff has determined that the proposed alternative measures for protection against radiological sabotage meet "the same high assurance objective," and "the general performance requirements" of the regulation and that "the overall level of system performance provides protection against radiological sabotage equivalent" to that which would be provided by the regulation.

Accordingly, the Commission has determined that, pursuant to 10 CFR 73.5, an exemption is authorized by law, will not endanger life or property or common defense and security, and is otherwise in the public interest. Therefore, the Commission hereby grants Consolidated Edison Company of New York, Inc. an exemption from those requirements of 10 CFR 73.55(d)(5) relating to the returning of picture badges/keycards upon exit from the protected area such that individuals not employed by the licensee, i.e., contractors, who are authorized unescorted access into the protected area, can take their badges/keycards offsite.

Pursuant to 10 CFR 51.32, the Commission has determined that the granting of this exemption will have no significant impact on the quality of the human environment (60 FR 56357). This exemption is effective upon issuance.

Dated at Rockville, Maryland, this 1st day of November 1995.

For the Nuclear Regulatory Commission.

Steven A. Varga,

*Director, Division of Reactor Projects—I/II,
Office of Nuclear Reactor Regulation.*

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[Docket No. 50-155]

Consumers Power Company (Big Rock Point Plant); Exemption

I

Consumers Power Company (CPCo, the licensee) is the holder of Facility Operating License No. DPR-6 which authorizes operation of the Big Rock Point Plant. The facility consists of a boiling water reactor located at the licensee's site in Charlevoix County, Michigan. The license provides, among other things, that the facility is subject to all rules, regulations, and orders of the Nuclear Regulatory Commission (the Commission) now or hereafter in effect.

II

Pursuant to 10 CFR 50.12(a), the NRC may grant exemptions from the requirements of the regulations (1) which are authorized by law, will not present an undue risk to the public health and safety, and are consistent with the common defense and security; and (2) where special circumstances are present.

Section 50.54(o) of 10 CFR requires that all licensees meet the requirements of Appendix J to Part 50—Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors. Paragraph III.D.2(b)(i) of Appendix J to 10 CFR Part 50 requires that containment air locks be tested at an internal pressure not less than peak pressure (P_a), which is 23 psig for Big Rock Point.

III

By letter dated October 4, 1994, as supplemented September 27, 1995, Consumers Power Company (the licensee) requested an exemption from the Appendix J requirement to test the air lock (escape lock) at P_a . Currently, the containment emergency (or escape) air lock at Big Rock Point is tested at a pressure of 2 psig. Therefore, the explicit requirement of paragraph III.D.2(b)(i) of Appendix J is not met. The requested exemption is required because of the emergency air lock manufacturer's restrictions on internal pressurization and the Big Rock Point design which necessitates frequent personnel entries. The licensee stated