

earthquakes, tornados, hurricanes, tornado missiles, and floods. To prevent water moderation, there is a drain at the low point of the vault to remove water in the vault to prevent accumulation of water within the NFV and no fuel is placed in the vault if there is water in the vault. The licensee also has procedures to prevent the introduction of an optimum moderation inside the vault (e.g., using pressurized water fire extinguishers instead of foam for combating fires around fuel) which could decrease the subcriticality margin to a value greater than the design value of 0.05. Although the Technical Specifications for Grand Gulf do not specifically limit the enrichment of the fuel onsite including the NFV, the k-effective for spent fuel or new fuel in the fuel racks and submerged in water is limited to 0.95 by the Technical Specifications and the enrichment of the fuel onsite is limited because the k-effective for the NFV is not allowed to be greater than 0.95. The fuel enrichment is a contributor to the value of k-effective. Therefore, the design of the NFV will preclude inadvertent criticality of the new fuel in the vault.

Therefore, based on the licensee's letters and the staff's evaluation, the Commission concludes that good cause exists for granting an exemption to the criticality monitoring requirements of 10 CFR 70.24(a) in storage areas for (1) in-core instrumentation detectors which are not in use and (2) unirradiated fuel stored in the NFV. Based on the information provided by the licensee, there is reasonable assurance that the nuclear instrumentation and unirradiated fuel will remain subcritical during handling and storage in areas where critically accident monitors required by 10 CFR 70.24(a) are not present. Additionally, all fuel storage and handling areas will continue to be monitored to detect conditions that may result in excessive radiation levels as required by General Design Criterion 63.

#### IV

For the foregoing reasons, pursuant to 10 CFR 70.24(d), the NRC staff has determined that good cause has been shown for granting an exemption to the criticality monitoring requirements of 10 CFR 70.24(a).

Accordingly, the Commission has determined that, pursuant to 10 CFR 70.14, an exemption is authorized by law, will not endanger life or property or common defense and security, and is otherwise in the public interest. Therefore, with the total amount of SNM contained in the in-core nuclear instruments less than a critical mass, as defined by Section 1.1 of Regulatory

Guide 10.3 (Revision 1, dated April 1977), with the unirradiated fuel assemblies only removed from the NRC-approved metal containers in areas where criticality monitors are present, and with administrative controls to prevent optimum moderation of the unirradiated fuel in the NFV, the Commission hereby grants Entergy Operations, Inc. an exemption from the criticality monitoring requirements of 10 CFR 70.24(a) for the storage of not-in-use in-core nuclear instrumentation and of unirradiated fuel in the NFV.

Pursuant to 10 CFR 51.32, the Commission has determined that the granting of this exemption will have no significant impact on the quality of the human environment (62 FR 55837). This exemption is effective upon issuance.

Dated at Rockville, Maryland, this 31st day of October 1997.

For the Nuclear Regulatory Commission.

**Samuel J. Collins,**

*Director, Office of Nuclear Reactor Regulation.*

[FR Doc. 97-29343 Filed 11-5-97; 8:45 am]

BILLING CODE 7590-01-P

### NUCLEAR REGULATORY COMMISSION

#### Correction to Biweekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Consideration

On October 22, 1997, the **Federal Register** published a Biweekly Notice of Applications and Amendments to Operating Licenses Involving No Significant Hazards Consideration. On page 54881, under Baltimore Gas and Electric Company, information from another notice was electronically merged with this notice causing an inaccurate publication. A copy of the notice, in its entirety, as it should have appeared follows:

*Baltimore Gas and Electric Company, Docket Nos. 50-317 and 50-318, Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2, Calvert County, Maryland.*

*Date of application for amendments:* March 28, 1996, as supplemented November 20, 1996, and July 31, 1997.

*Brief description of amendments:* The amendments reduce the moderator temperature coefficient limit shown on Technical Specification Figure 3.1.1-1. This proposed change is necessary to support changes in the safety analyses made to accommodate a larger number of plugged steam generator tubes for future operating cycles.

*Date of issuance:* October 2, 1997.

*Effective date:* As of the date of issuance to be implemented within 30 days.

*Amendment Nos.:* 222 and 198.

*Facility Operating License Nos. DPR-53 and DPR-69:* Amendments revised the Technical Specifications.

*Date of initial notice in Federal Register:* May 8, 1996 (61 FR 20843).

The November 20, 1996, and July 31, 1997, letters provided clarifying information that did not change the initial proposed no significant hazards consideration determination.

The Commission's related evaluation of these amendments is contained in a Safety Evaluation dated October 2, 1997.

No significant hazards consideration comments received: No.

*Local Public Document Room*

*location:* Calvert County Library, Prince Frederick, Maryland 20678.

Dated at Rockville, Maryland, this 31st day of October 1997.

For the Nuclear Regulatory Commission.

**Elinor G. Adensam,**

*Acting Director, Division of Reactor Projects III/V, Office of Nuclear Reactor Regulation.*

[FR Doc. 97-29345 Filed 11-5-97; 8:45 am]

BILLING CODE 7590-01-P

### NUCLEAR REGULATORY COMMISSION

#### Consolidated Guidance About Materials Licenses: Applications for Sealed Source and Device Evaluation and Registration, Availability of Draft NUREG

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Notice of availability and request for comments.

**SUMMARY:** The Nuclear Regulatory Commission is announcing the availability of and requesting comment on draft NUREG-1556, Vol. 3, "Consolidated Guidance about Materials Licenses: Applications for Sealed Source and Device Evaluation and Registration," dated September 1997.

NRC is consolidating and updating numerous guidance documents into a series of program specific guidance documents to be published in a NUREG format. All NUREGs in the series will carry the number and title: NUREG 1556, Volume X, "Consolidated Guidance About Materials Licenses." Each specific guidance document will have an identifying volume number, and the title of the specific guidance will appear as the sub-title of the NUREG. This draft NUREG is the third guidance document to be published in