

Further information and self-explanatory forms for the purpose of submitting this data may be obtained by writing to the U.S. Department of Labor, Employment Standards Administration, Wage and Hour Division, Division of Wage Determinations, 200 Constitution Ave., N.W., Room S-3014, Washington, DC 20210.

### Modifications to General Wage Determination Decisions

The number of decisions listed in the Government Printing Office document entitled "General Wage Determinations Issued Under the Davis-Bacon and Related Acts" being modified are listed by Volume and State. Dates of publication in the **Federal Register** are in parentheses following the decisions being modified.

#### Volume I

New Jersey  
NJ980002 (Feb. 13, 1998)

#### Volume II

Pennsylvania  
PA980023 (Feb. 13, 1998)  
PA980024 (Feb. 13, 1998)  
PA980029 (Feb. 13, 1998)  
PA980040 (Feb. 13, 1998)  
PA980052 (Feb. 13, 1998)  
PA980060 (Feb. 13, 1998)  
PA980063 (Feb. 13, 1998)

#### West Virginia

WV980002 (Feb. 13, 1998)  
WV980003 (Feb. 13, 1998)  
WV980006 (Feb. 13, 1998)

#### Volume III

Florida  
FL980045 (Feb. 13, 1998)  
Georgia  
GA980083 (Feb. 13, 1998)  
North Carolina  
NC980050 (Feb. 13, 1998)  
South Carolina  
SC980036 (Feb. 13, 1998)

#### Volume IV

Illinois  
IL980018 (Feb. 13, 1998)  
IL980033 (Feb. 13, 1998)  
IL980035 (Feb. 13, 1998)  
IL980036 (Feb. 13, 1998)  
IL980039 (Feb. 13, 1998)  
IL980044 (Feb. 13, 1998)  
IL980056 (Feb. 13, 1998)

#### Michigan

MI980007 (Feb. 13, 1998)  
MI980066 (Feb. 13, 1998)  
MI980068 (Feb. 13, 1998)  
MI980071 (Feb. 13, 1998)  
MI980074 (Feb. 13, 1998)  
MI980078 (Feb. 13, 1998)  
MI980080 (Feb. 13, 1998)

#### Volume V

Oklahoma  
OK980013 (Feb. 13, 1998)  
OK980014 (Feb. 13, 1998)  
OK980016 (Feb. 13, 1998)  
OK980017 (Feb. 13, 1998)  
OK980028 (Feb. 13, 1998)

OK980034 (Feb. 13, 1998)  
OK980035 (Feb. 13, 1998)  
OK980036 (Feb. 13, 1998)  
OK980037 (Feb. 13, 1998)  
OK980038 (Feb. 13, 1998)  
OK980043 (Feb. 13, 1998)

#### Volume VI

Washington  
WA980001 (Feb. 13, 1998)  
WA980002 (Feb. 13, 1998)  
Wyoming  
WY980009 (Feb. 13, 1998)

#### Volume VII

California  
CA980009 (Feb. 13, 1998)  
CA980029 (Feb. 13, 1998)

### General Wage Determination Publication

General Wage Determinations issued under the Davis-Bacon and related Acts, including those noted above, may be found in the Government Printing Office (GPO) document entitled "General Wage Determinations Issued Under the Davis-Bacon and Related Acts." This publication is available at each of the 50 Regional Government Depository Libraries and many of the 1,400 Government Depository Libraries across the country.

The General Wage Determinations issued under the Davis-Bacon and related Acts are available electronically by subscription to the FedWorld Bulletin Board System of the National Technical Information Service (NTIS) of the U.S. Department of Commerce at (703) 487-4630.

Hard-copy subscriptions may be purchased from: Superintendent of Documents, U.S. Government Printing Office, Washington, DC 20402, (202) 512-1800.

When ordering hard-copy subscription(s), be sure to specify the State(s) of interest, since subscriptions may be ordered for any or all of the seven separate volumes, arranged by State. Subscriptions include an annual edition (issued in January or February) which includes all current general wage determinations for the States covered by each volume. Throughout the remainder of the year, regular weekly updates are distributed to subscribers.

Signed at Washington, DC this 20th day of March 1998.

**Carl Poleskey,**

*Chief, Branch of Construction Wage Determinations.*

[FR Doc. 98-7719 Filed 3-26-98; 8:45 am]

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### NATIONAL AERONAUTICS AND SPACE ADMINISTRATION

[Notice (98-043)]

### NASA Advisory Council (NAC), Aeronautics and Space Transportation Technology Advisory Committee, Task Force on NASA's Aviation Environmental Compatibility Research; Meeting

AGENCY: National Aeronautics and Space Administration.

ACTION: Notice of meeting.

**SUMMARY:** In accordance with the Federal Advisory Committee Act, Pub. L. 92-463, as amended, the National Aeronautics and Space Administration announces a forthcoming meeting of the NASA Advisory Council, Aeronautics and Space Transportation Technology Advisory Committee, Task Force on NASA's Aviation Environmental Compatibility Research.

**DATES:** Wednesday, April 22, 1998, 8:00 a.m. to 5:00 p.m.

**ADDRESSES:** National Aeronautics and Space Administration, Room 9H40, 300 E Street, SW., Washington, DC 20546.

**FOR FURTHER INFORMATION CONTACT:** Ms. Darlene Boykins, Office of Aeronautics and Space Transportation Technology, National Aeronautics and Space Administration, Washington, DC 20546 (202/358-4743).

**SUPPLEMENTARY INFORMATION:** The meeting will be open to the public up to the seating capacity of the room. The purpose of the meeting is to review actions related to the Task Force Charter listed below:

- Based on examining past application of NASA research, recommend ways to improve effectiveness of environmental technology transfer.
  - Evaluate process being used to assess and recommend NASA research plans in noise and emissions relative to the "Three Pillars" environmental goals.
  - Recommend ways to ensure the appropriate use of research in regulatory considerations.
  - Recommend ways of improving the relationship of NASA with the air carrier community, aircraft and engine manufacturers, other environmental research and technology organizations, and regulatory agencies with regard to environmental research and technology.
  - Identify critical interdependencies of environmental goals with the other related "Three Pillar" goals.
- It is imperative that the meeting be held on these dates to accommodate the

scheduling priorities of the key participants.

**Matthew M. Crouch,**

*Advisory Committee Management Officer,  
National Aeronautics and Space  
Administration.*

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## NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-282 and 50-306]

### **Northern States Power Co.; Notice of Consideration of Issuance of Amendments to Facility Operating Licenses, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing**

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of amendments to Facility Operating Licenses DPR-42 and DPR-60 issued to Northern States Power Company (the licensee) for operation of the Prairie Island Nuclear Generating Plant, Units 1 and 2, located in Goodhue County, Minnesota.

The proposed amendments would (1) update the Technical Specification heatup and cooldown rate curves and extend their reactor fluence limit from the current 20 effective full power years (EFPY) to a new value of 35 EFPY, (2) incorporate into Technical Specifications the use of a Pressure and Temperature Limits Report (PTLR), and (3) change the power-operated relief valves (PORVs) temperature requirement for operability.

Before issuance of the proposed license amendments, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed amendment[s] will not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed change to update the Prairie Island pressure and temperature limits curves and the Over Pressure Protection System (OPPS) setpoints for reactor vessel fluence to 35 EFPY is based upon measurements and calculations that were performed in accordance with an NRC approved methodology for performing reactor vessel fracture analysis to meet 10CFR50 Appendix G and H requirements. These calculations made application of American Society of Mechanical Engineers (ASME) Code Case N-514, "Low Temperature Overpressure Protection," in determining the acceptable OPPS setpoint for Prairie Island Units 1 and 2. This permits the OPPS pressure relief setpoint to be established such that the maximum pressure at the reactor vessel material's most limiting location is limited to 110% of the pressure determined to satisfy ASME Section XI, Appendix G, Article G-2215. As detailed in the exemption request to apply this ASME Code Case, the development of the Appendix G pressure/temperature limit curves incorporates numerous conservatisms. For this reason the ASME code committee approved this code case. Application of this code case with the approved methodology does not produce a significant increase in the probability or magnitude of brittle fracture of the reactor vessel.

The proposed change to relocate the pressure and temperature limits curves and the Over Pressure Protection System (OPPS) setpoints to a Pressure and Temperature Limits Report is an administrative change. It does not affect any system which is a contributor to initiating events for previously evaluated anticipated operational occurrences and therefore does not involve any significant increase in the probability or consequence of an accident previously evaluated.

The proposed change in PORV operability temperature from 310 °F to a new value of 350 °F does not affect any system which is a contributor to initiating events for previously evaluated anticipated operational occurrences and therefore does not involve any significant increase in the probability or consequence of an accident previously evaluated.

2. The proposed amendment[s] will not create the possibility of a new or different kind of accident from any accident previously analyzed.

The proposed change to update the Prairie Island pressure and temperature limits curves and the Over Pressure Protection System (OPPS) setpoints for a reactor fluence limit of 35 EFPY does not introduce a new mode of operation or testing, or make physical changes to the plant. (The new Technical Specification requirement to isolate the accumulators whenever the RCS [reactor coolant system] temperature is less than the OPPS enable temperature does not introduce a new mode of operation since Unit Shutdown procedures close the accumulator discharge valves and tag out their breakers when RCS pressure falls below

1000 psig.) The general methods employed to develop this change are well understood and have been previously reviewed and approved. Updating the operating restrictions, OPPS setpoints, and reactor fluence limit for operation do not create a possibility of a new or different kind of accident from those previously analyzed.

The proposed change to relocate the pressure and temperature limits curves and the Over Pressure Protection System (OPPS) setpoints to a Pressure and Temperature Limits Report is an administrative change. The proposed change does not alter the design, function, or operation of any plant component, therefore a possibility of a new or different kind of accident from those previously analyzed has not [been] created.

The proposed change in the PORV operability temperature from 310 °F to a new value of 350 °F does not involve a physical alteration of the plant. Since no new or different type of equipment will be installed, this change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. The proposed amendment[s] will not involve a significant reduction in the margin of safety.

Although neutron irradiation reduces the material fracture toughness of the reactor vessel, deterministic analyses have demonstrated that proposed P/T [pressure/temperature] curves, OPPS setpoints and reactor vessel fluence limits for operation will preserve the required margin of safety in the RCS boundary during postulated low temperature pressurization events.

The proposed change to use the PTLR is administrative in nature and does not impact the operation of the Prairie Island Nuclear Generating Plant in a manner that would result in any significant reduction in any margin of safety.

The proposed change in the PORV operability temperature from 310 °F to a new value of 350 °F does not impact any systems that are relied upon for core cooling or RCS pressure relief at RCS temperatures below 350 °F. Setting the PORV operability temperature back to 350 °F aligns the PORVs with the Pressurizer Safety Valve operability requirement so the PORVs are still available to limit challenges to the Pressurizer Safety Valve settings during conditions of higher RCS pressure and energy when pressure surges become more significant. (In Amendment[s] 91/84 this temperature was changed for operational flexibility from its previous value of 350 °F to a new value of 310 °F to be coincident with the OPPS Enable Temperature. This change was not done to establish a larger margin of safety.) For RCS temperatures below 350 °F both the pressure and core energy are sufficiently decreased that pressure surges become less significant. For RCS temperatures below 350 °F the RHR [residual heat removal] system is capable of removing the reactor decay heat and thereby controlling RCS pressure and temperature. In the unlikely event that a significant pressure surge were to occur in this temperature range with neither RHR nor the PORVs in service, one pressurizer safety valve would be operable to mitigate potential overpressure transients. Thus this change does not involve