ADDRESS: Lyndon B. Johnson Space Center, National Aeronautics and Space Administration, Building 1, Room 920L, Houston, TX 77058–3696.

FOR FURTHER INFORMATION CONTACT: Mr. Dennis McSweeney, Code IH, National Aeronautics and Space Administration, Washington, DC 20546–0001, 202/358–4556.

SUPPLEMENTARY INFORMATION: This meeting will be open to the public up to the seating capacity of the room. The agenda for the meeting is as follows:

—Review the readiness of the STS–91 Shuttle-Mir Rendezvous and Docking Mission.

It is imperative that the meeting be held on these dates to accommodate the scheduling priorities of the key participants. Visitors will be requested to sign a visitors register.

Dated: April 22, 1998.

Mathew M. Crouch,

Advisory Committee Management Officer, National Aeronautics and Space Administration.

[FR Doc. 98–11267 Filed 4–27–98; 8:45 am]

BILLING CODE 7510–01–M

NATIONAL SCIENCE FOUNDATION

Long Term Durability of Materials and Structures; Special Emphasis Panel in Civil and Mechanical Systems; Notice of Meeting

In accordance with the Federal Advisory Committee Act Public Law 92–463, as amended, the National Science Foundation announces the following meeting:

Name: Long Term Durability of Materials and Structures (1205).

Date & Time: May 14, 18, and 19, 1998; 8:30 a.m. to 5:00 p.m.

Place: NSF, 4201 Wilson Boulevard, Rooms 310, 375, 410, 530, 580, 1020 and 1295 Arlington, Virginia 22230.

Contact Person: Dr. Jorn Larsen-Basse, Program Director, Control, Materials and Mechanics Cluster, Division of Civil and Mechanical Systems, Room 545, NSF, 4201 Wilson Blvd., Arlington, VA 22230. 703/306–1361, x 5073.

Purpose of Meeting: To provide advice and recommendations concerning proposals submitted to NSF for financial support.

Agenda: To review and evaluate Long-Term Durability of Materials and Structures research proposals as part of the selection process for awards.

Reason for Closing: The proposals being reviewed include information of a proprietary or confidential nature, including technical information; financial data, such as salaries; and personal information concerning individuals associated with the proposals. These matters are exempt under 5 U.S.C. 552b(c) (4) and (6) of the Government Sunshine Act.

Dated: April 23, 1998.

M. Rebecca Winkler,

Committee Management Officer.

[FR Doc. 98–11202 Filed 4–27–98; 8:45 am]

BILLING CODE 7555–01–M

NATIONAL SCIENCE FOUNDATION

Advisory Committee for Social, Behavioral, and Economic Sciences; Meeting

In accordance with the Federal Advisory Committee Act (Pub. L. 92–463, as amended), the National Science Foundation announces the following meeting:

Name: Advisory Committee for Social, Behavioral, and Economic Sciences (1171).

Date and Time: May 20, 1998; 9 a.m.–5 p.m.; May 21, 1998; 9 a.m.–3:30 p.m.

Place: NSF, Room 1235, NSF, 4201 Wilson Blvd., Arlington, Va. 22230.

Type of Meeting: Open.

Contact Person: Ms. Catherine J. Hines, Executive Secretary; Directorate for Social, Behavioral, and Economic Sciences, NSF, Suite 905; 4201 Wilson Blvd., Arlington, Va. 22230. Telephone: (703) 306–1741.

Minutes: May be obtained from the contact person listed above.

Purpose of Meeting: To provide advice and recommendations to the National Science Foundation on major goals and policies pertaining to SBE programs and activities.

Agenda: Discussions on issues, role, and future direction of the NSF Directorate for Social, Behavioral and Economic Sciences.

Dated: April 23, 1998.

M. Rebecca Winkler,

Committee Management Officer.

[FR Doc. 98–11201 Filed 4–27–98; 8:45 am]

BILLING CODE 7555–01–M

NATIONAL TRANSPORTATION SAFETY BOARD

Sunshine Act Meeting

TIME AND DATE: 9:30 a.m., Tuesday, May 5, 1998.

PLACE: NTSB Board Room, 5th Floor, 490 L'Enfant Plaza, SW., Washington, DC 20594.

STATUS: Open.

MATTERS TO BE CONSIDERED:

5299D—''Most Wanted'' Safety Recommendation Program Status Report and Suggested Modification

6773A—Marine Special Investigation Report—Postaccident Alcohol and Other Drug Testing in the Marine Industry and the Ramming of the Portland South Portland Bridge at Portland, Maine, by the Liberian Tankship Julie N on September 27, 1996

6996—Highway/Hazardous Material Summary Report—Collision and Fire of Tractor/Cargo Tank Semitrailer and Passenger Vehicle, October 9, 1997

NEWS MEDIA CONTACT: Telephone (202) 314–6100.

FOR MORE INFORMATION CONTACT: Rhonda Underwood, (202) 314–6065.

Dated: April 24, 1998.

Rhonda Underwood,

Federal Register Liaison Officer.

[FR Doc. 98–11442 Filed 4–24–98; 3:43 pm]

BILLING CODE 7533–01–M

NUCLEAR REGULATORY COMMISSION

Agency Information Collection Activities: Submission for OMB Review; Comment Request

AGENCY: U.S. Nuclear Regulatory Commission (NRC).

ACTION: Notice of the OMB review of information collection and solicitation of public comment.

SUMMARY: The NRC has recently submitted to OMB for review the following proposal for the collection of information under the provisions of the Paperwork Reduction Act of 1995 (44 U.S.C. Chapter 35). The NRC hereby informs potential respondents that an agency may not conduct or sponsor, and that a person is not required to respond to, a collection of information unless it displays a currently valid OMB control number.

1. *Type of submission, new, revision, or extension:* Revision.

2. *The title of the information collection:* ''Licensee Event Report''.

3. *The form number if applicable:* NRC Form 366.

4. *How often the collection is required:* On occasion.

5. *Who will be required or asked to report:* Holders of Operating Licenses for Commercial Nuclear Power Plants.

6. *An estimate of the number of responses:* 1,600 per year.

7. *The estimated number of annual respondents:* 109 Holders of Operating Licenses for Commercial Nuclear Power Plants.

8. *An estimate of the total number of hours needed annually to complete the requirement or request:* Approximately 50 hours per response. The total industry burden is 80,000 hours.

9. *An indication of whether Section 3507(d), Pub. L. 104–13 applies:* Not Applicable

10. *Abstract:* NRC collects reports of operational events at commercial nuclear power plants in order to incorporate lessons of that experience in

the licensing process and to feed back the lessons of that experience to the nuclear industry.

A copy of the final supporting statement may be viewed free of charge at the NRC Public Document Room, 2120 L Street, NW (lower level), Washington, DC. OMB clearance requests are available at the NRC worldwide web site (http://www.nrc.gov) under the FedWorld collection link on the home page tool bar. The document will be available on the NRC home page site for 60 days after the signature date of this notice.

Comments and questions should be directed to the OMB reviewer by May 28, 1998: Erik Godwin, Office of Information and Regulatory Affairs (3150–0104), NEOB–10202, Office of Management and Budget, Washington, DC 20503.

Comments can also be submitted by telephone at (202) 395–3084.

The NRC Clearance Officer is Brenda Jo Shelton, 301–415–7233.

Dated at Rockville, Maryland, this 21st day of April 1998.

For the Nuclear Regulatory Commission.

Brenda Jo Shelton,

NRC Clearance Officer, Office of the Chief Information Officer.

[FR Doc. 98–11245 Filed 4–27–98; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50–293]

Boston Edison Company; Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. DPR–35, issued to Boston Edison Company (BECo/the licensee), for operation of the Pilgrim Nuclear Power Station located in Plymouth, Massachusetts.

The proposed amendment would modify Technical Specification (TS) Section 3.6.A.1 to remove the requirement that the reactor vessel flange and adjacent shell differential temperature be monitored during heatup and cooldown events and also removes the 145 degrees Fahrenheit differential temperature limit.

By letter dated April 8, 1998, the licensee requested that the proposed TS change be reviewed under exigent circumstances. A normal plant cooldown under current TS requirements would require monitoring reactor vessel shell flange temperature to maintain the vessel flange to adjacent vessel shell differential temperature at less than 145 degrees Fahrenheit. However, the current condition of the vessel shell flange thermocouples prohibits accurate monitoring of the metal surface temperature to meet this TS requirement. The thermocouples are considered inoperable due to inconsistencies in their readouts. Because the need for plant shutdown and cooldown cannot be forecasted in advance, BECo has requested review of the submitted change under exigent circumstances to avoid a future short-notice request and possible violation of current TS requirements. BECo has made a good faith effort to prepare the proposed license amendment for NRC approval as expeditiously as practicable.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

Pursuant to 10 CFR 50.91(a)(6) for amendments to be granted under exigent circumstances, the NRC staff must determine that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

a. The proposed amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The recent analysis, Ref.[10], [see application dated March 25, 1998] has shown design and licensing bases for reactor vessel integrity will be maintained, and results supporting the T. S. change show the conclusions reached remain unchanged from previous conclusions reached in Ref.[3] [see application dated March 25, 1998] and as described in the [final safety analysis report] FSAR, Ref.[1] [see application dated March 25, 1998]. Structural integrity for design basis loading conditions is assured, based on the results of Ref.[10] [see application dated March 25, 1998]. The ability to control plant heatup and cooldown rates has been shown by analysis to be unaffected by the removal of this T. S. requirement. This has been confirmed by initial startup testing results and the past 25 years of service.

b. The proposed amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated.

T/C's [thermocouples] used to monitor reactor vessel flange to adjacent shell DT [differential temperature] are used only during normal startup and shutdown conditions, and removal of the T. S. requirement to monitor this differential temperature will have no affect on the design basis accident conditions. Moderator temperature and pressure are monitored and, in the event fluid ramp rates exceed design basis requirements, an evaluation must be performed to determine the effect on structural integrity of the reactor vessel and components. ASME Code Section XI, Appendix E, Ref. [11] [see application dated March 25, 1998], provides a method for evaluating an operating event that causes excursion outside these limits.

c. The proposed amendment does not involve a significant reduction in the margin of safety.

Stress and fracture toughness calculations, Ref.[10] [see application dated March 25, 1998], have shown removal of the T. S. DT requirement will not increase levels above the conservative design basis limits previously established in the analysis of record, Ref.[3] [see application dated March 25, 1998], or those stated in the FSAR, Ref.[1] [see application dated March 25, 1998]. The loadings used to determine stresses are the same provided by the original equipment designer and manufacturer. The calculated stress levels and fatigue damage assessment for the existing condition are essentially unchanged from the values reported in the reactor vessel analysis of record, Ref.[3] [see application dated March 25, 1998]. The results of the recent analysis, Ref.[10] [see application dated March 25, 1998], show that the margins of safety, as defined in the bases for the Pilgrim T. S. and the FSAR, are not reduced and vessel integrity will be maintained during all normal and transient conditions previously analyzed and reported in the FSAR.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 14 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 14-day notice period. However, should circumstances change during the notice period, such that failure to act in a timely way would