

NSF will support, we contact each institute director to ask for a list of up to 5 nominations to be considered for NSF travel support.

- EHR Contact with the Individuals Nominated

Each individual who is nominated by a director will be sent the rules of eligibility, information about the amount of funding available, and the forms (NSF Form 1379, giving our Finance Office electronic banking information; NSF Form 1310 (already cleared), and NSF Form 192 (Application for International Travel Grant)) necessary for our application process.

- The Funding Process

Once an applicant has been selected to receive NSF travel award support, his or her application is sent to our Finance office for funding. They electronically transfer the amount of \$1000 into the bank or other financial institution account identified by the awardee.

Our plan is to have the \$1000 directly deposited into the awardee's account prior to the purchase of their airline ticket. An electronic message to the awardee states the NSF is providing support in the amount of \$1000 for transportation and miscellaneous expenses. The letter also states that the award is subject to the condition in F.L. 27, *Attachment to International Travel Grant*, which states the U.S. flag-carrier policy.

As a follow-up, each ASI director may be asked to verify whether all NSF awardees attended the institute. If an awardee is identified as not utilizing the funds as prescribed, we contact the awardee to retrieve the funds. However, if our efforts are not successful, we will forward the awardee's name to DGA, which has procedures to deal with that situation.

We also ask the awardee to submit a final report on an NSF Form 250, which we provide as an attachment to the electronic award message.

- Selection of Awardees

The criteria used to select NSF Advanced Study Institute travel awardees are as follows:

1. The priority of selection is by the status level of the applicant:
 - (a) Advanced graduate student, or
 - (b) Recent post-doc (Ph.D. received no earlier than three years before the ASI).
 - (c) New faculty with Ph.D.'s received no earlier than three years before the ASI).

2. We shall generally follow the order of the nominations, listed by the director of the institute, within priority level.

3. Those who have not attended an ASI in the past will have a higher priority than those who have.

4. Nominees from different institutions and research groups have higher priority than those from the same institution or research group. (Typically, no more than one person is invited from a school or from a research group.)

Use of the Information: For NSF Form 192, information will be used in order to verify eligibility and qualifications for the award.

For NSF Form 250, information will be used to verify attendance at Advanced Study Institute and will be included in Division annual report.

Estimate of Burden: Form 192—1.5 hours; Form 250—2 hours.

Respondents: Individuals.

Estimated Number of Responses per Award: 150 responses, broken down as follows: For NSF Form 250, 75 respondents; for NSF Form 192, 75 respondents.

Estimated Total Annual Burden on Respondents: 262.5 hours, broken down by 150 hours for NSF Form 250 (2 hours per 75 respondents) and 112.5 hours for NSF Form 192 (1.5 hours per 75 respondents).

Dated: January 8, 2002.

Suzanne H. Plimpton,

Reports Clearance Officer.

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NUCLEAR REGULATORY COMMISSION

[Docket No. 50-286]

Entergy Nuclear Operations, Inc.; Notice of Withdrawal of Application for Amendment to Facility Operating License

The U.S. Nuclear Regulatory Commission (the Commission) has granted the request of Entergy Nuclear Operations, Inc. (the licensee) to withdraw its February 14, 2001, application for proposed amendment to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3 (IP3), located in Westchester County, New York.

The proposed amendment would have revised the Technical Specifications to extend the allowed outage time for the emergency diesel generators and the associated fuel oil storage tanks from 72 hours to 14 days on a one-time basis.

The Commission had previously issued a Notice of Consideration of Issuance of Amendment published in the **Federal Register** on March 21, 2001

(66 FR 15922). However, by letter dated December 5, 2001, the licensee withdrew the proposed change.

Further details with respect to this action, see the application for amendment dated February 14, 2001, as supplemented on July 25, 2001, and the licensee's letter dated December 5, 2001, which withdrew the application for license amendment. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the Agencywide Documents Access and Management Systems (ADAMS) Public Electronic Reading Room on the internet at the NRC Web site, <http://www.nrc.gov/reading-rm/adams.html>. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS, should contact the NRC Public Document Room (PDR) Reference staff by telephone at 1-800-397-4209, 301-415-4737 or by email to pdr@nrc.gov.

Dated at Rockville, Maryland, this 8th day of January 2002.

For the Nuclear Regulatory Commission.

Patrick D. Milano,

Sr. Project Manager, Section 1, Project Directorate, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 02-847 Filed 1-11-02; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[Docket No. 50-289]

Amergen Energy Company, LLC; Three Mile Island Nuclear Station, Unit 1 Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of an exemption from the requirements of Title 10 of the Code of Federal Regulations (10 CFR), section 50.44; 10 CFR part 50, appendix A, General Design Criterion 41; and 10 CFR part 50, appendix E, section VI, for Facility Operating License No. DPR-50 issued to AmerGen Energy Company, LLC, (the licensee), for operation of the Three Mile Island Nuclear Station, Unit 1 (TMI-1), located in Dauphin County, Pennsylvania. Therefore, as required by 10 CFR 51.21, the NRC is issuing this environmental assessment and finding of no significant impact.

Environmental Assessment

Identification of the Proposed Action

The proposed action would exempt TMI-1 from certain requirements of 10 CFR 50.44; 10 CFR part 50, appendix A, General Design Criterion 41; and part 10 CFR 50, appendix E, section VI, pertaining to the hydrogen control system requirements (i.e., containment post-accident hydrogen monitors, recombiners, and hydrogen purge system); and remove these requirements from the TMI-1 design basis. The licensee's request for an exemption from the functional requirement for hydrogen monitoring is not being approved. The NRC staff's position, with respect to each of the licensee's specific exemption requests, will be documented in the exemption. Consequently, this environmental assessment addresses only the exemption from the requirements related to the recombiners and the hydrogen purge system and the removal of these requirements from the TMI-1 design basis.

The proposed action is in accordance with the licensee's letter dated September 20, 2000, as supplemented by letters dated August 2 and September 28, 2001.

The Need for the Proposed Action

The proposed exemption from the requirements pertaining to recombiners and the hydrogen purge system, and their associated removal from the design basis, would improve the safety focus at TMI-1 during an accident, and provide for a more effective and efficient method of maintaining adequate protection of public health and safety by simplifying the Emergency Plan and Emergency Plan Implementing Procedures. This would reduce the operators' post-accident burden and allow them to give higher priority to more important safety functions following postulated plant accidents.

Environmental Impacts of the Proposed Action

The NRC has completed its evaluation of the proposed action and concludes, as set forth below, that there are no environmental impacts associated with the removal of the recombiners and hydrogen purge system from the TMI-1 design basis.

The proposed action will not significantly increase the probability or consequences of accidents, no changes are being made in the types of effluents that may be released off site, and there is no significant increase in occupational or public radiation exposure. Therefore, there are no significant radiological environmental

impacts associated with the proposed action.

With regard to potential nonradiological impacts, the proposed action does not involve any historic sites. It does not affect nonradiological plant effluents and has no other environmental impact. Therefore, there are no significant nonradiological environmental impacts associated with the proposed action.

Accordingly, the NRC concludes that there are no significant environmental impacts associated with the proposed action.

Alternatives to the Proposed Action

As an alternative to the proposed action, the NRC staff considered denial of the proposed action (i.e., the "no-action" alternative). Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

The action does not involve the use of any resources not previously considered in the Final Environmental Statement for TMI-1, dated December 1972.

Agencies and Persons Consulted

On December 11, 2001, the staff consulted with the Pennsylvania State official, Mr. Michael Murphy of the Pennsylvania Bureau of Radiation Protection, regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

On the basis of the environmental assessment, the NRC concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the NRC has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated September 20, 2000, as supplemented by letters dated August 2 and September 28, 2001. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the Agencywide Documents Access and Management System (ADAMS) Public Library component on the NRC Web site, <http://www.nrc.gov> (the Public Electronic Reading Room). Persons who do not have access to ADAMS or who encounter problems in

accessing the documents located in ADAMS, should contact the NRC PDR Reference staff by telephone at 1-800-397-4209, or 301-415-4737, or by e-mail at pdr@nrc.gov.

Dated at Rockville, Maryland, this 8th day of January 2002.

For the Nuclear Regulatory Commission.

Timothy G. Colburn,

Senior Project Manager, Section 1, Project Directorate 1, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

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NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards: Subcommittee Meeting on Planning and Procedures; Notice of Meeting

The ACRS Subcommittee on Planning and Procedures will hold a meeting on January 24-26, 2002, Hawthorn Suites, 6435 Westwood Blvd., Orlando, Florida, in Conference Room Magnolia A.

The entire meeting will be open to public attendance, with the exception of a portion that may be closed pursuant to 5 U.S.C. 552(b)(2) and (6) to discuss organizational and personnel matters that relate solely to internal personnel rules and practices of ACRS, and information the release of which would constitute a clearly unwarranted invasion of personal privacy.

The agenda for the subject meeting shall be as follows:

Thursday, January 24, 2002—8:30 a.m. until the conclusion of business

The Subcommittee will discuss matters related to future plant designs, including: regulatory challenges associated with the licensing of future plant designs (e.g., Pebble Bed Modular Reactor and Gas Turbine Modular Helium Reactor); use of PRA and defense-in-depth concept for advanced reactor designs; and issues related to Westinghouse AP1000 design. Also, it will discuss the NRC Safety Research Program, including proposed advanced reactor research plan, new areas of research, and draft ACRS report to the Commission on the NRC Safety Research Program.

Friday, January 25, 2002—8:30 a.m. until the conclusion of business

The Subcommittee will discuss the use of formal decision analysis and the role of SAPHIRE Code in the risk-informed regulatory structure. Also, it will discuss matters associated with core power uprates, including: use of risk information in evaluating power