where the weather conditions change rapidly and dramatically. Vessels working in these high latitudes are subject to demanding and often dangerous conditions due to low temperatures, high winds, and rough seas. Failure to meet any of the technical requirements would have severe negative consequences for the project with regard to operational safety.

The market research for this exemption was done by the shipyard in the summer of 2010 and verified by the UAF project team in July 2010. As noted in UAF's request for this exemption, the shipyard performed market research by reviewing industry publications, the Internet (including the Marine Electronics Journal Web site) and contacting various electronic supply companies in order to assess whether there exists a domestic capability to provide a weather fax that meets the necessary requirements for safe and successful operation world-wide. Eighteen (18) potential vendors were identified with only six (6) manufacturing a weather fax. Of the six, only one (1) was a U.S. manufacturer. The shipyard then compared the existing product lines for compliance with the weather fax technical specifications and requirements as identified above. It was found that the one U.S. manufacturer did not make a unit that was stand-alone. Instead, the system uses a personal computer to provide both the human interface and printing capability of the weather charts. This requires the bridge watch to actively manage and interface with the system, which takes their attention from other navigational and operational duties. This distraction increases the likelihood of collision, grounding, failure to adequately monitor over-the-side science operations, and inadvertently sailing into dangerous weather conditions. Because of this, all modern ocean-going vessels have at least one stand-alone weather fax system.

The project's conclusion is there are no U.S. manufacturers who produce a suitable weather fax unit that meets all of the ARRV requirements so an exemption to the Buy American requirements is necessary.

In the absence of a domestic supplier that could provide a requirements-compliant weather fax, UAF requested that NSF issue a Section 1605 exemption determination with respect to the purchase of a foreign-supplied, requirements-compliant weather fax, so that the vessel will meet the specific design and technical requirements which, as explained above, are necessary for this vessel to be able to perform its mission safely and successfully. Furthermore, the shipyard's market research as verified by UAF indicated that a weather fax compliant with the ARRV's technical specifications and requirements is commercially available from foreign vendors within their standard product lines.

NSF's Division of Acquisition and Cooperative Support (DACS) and other NSF program staff reviewed the UAF exemption request submittal, found that it was complete, and determined that sufficient technical information was provided in order for NSF to evaluate the exemption request and to conclude that an exemption is needed and should be granted.

III. Exemption

On October 22, 2010, based on the finding that no domestically produced weather fax met all of the ARRV's technical specifications and requirements and pursuant to section 1605(b), the NSF Chief Financial Officer, in accordance with a delegation order from the Director of the agency, granted a limited project exemption of the Recovery Act's Buy American requirements with respect to the procurement of the marine weather fax.

Dated: December 23, 2010.

Lawrence Rudolph,

General Counsel.

[FR Doc. 2010–33044 Filed 12–30–10; 8:45 am]

BILLING CODE 7555–01–P

NUCLEAR REGULATORY COMMISSION

Programmatic Environmental Assessment and Final Finding of No Significant Impact for Exemptions From the Implementation Deadline for New Security Requirements

The U.S. Nuclear Regulatory Commission (NRC) has received and expects to receive exemption requests from several nuclear power reactor licensees. Each expected exemption request will be from the implementation date requirement of Title 10 of the Code of Federal Regulations (10 CFR) 73.55. The NRC is authorized to issue exemptions pursuant to 10 CFR 73.5. In accordance with 10 CFR 51.21, the NRC has performed a programmatic environmental assessment of these exemption requests. The NRC concluded that the proposed action constitutes administrative (timing) changes that would not impact the environmental resources near any specified nuclear power plant. Based upon the results of this programmatic environmental assessment, the NRC is issuing a finding of no significant impact.

Environmental Assessment

Identification of the Proposed Action

The proposed action may include issuing exemptions to nuclear power plant licensees for up to 40 nuclear power plant sites, all of which have already been granted plant-specific exemptions granting additional time to implement some of the new requirements of 10 CFR 73.55. These sites are:

Arkansas Nuclear One, Units 1 and 2
Beaver Valley Power Station, Unit Nos. 1 and 2
Browns Ferry Nuclear Plant, Units 1, 2, and 3
Brunswick Steam Electric Station, Units 1 and 2
Columbia Generating Station
Cooper Nuclear Station
Crystal River Unit 3 Nuclear Generating Plant
Davis-Besse Nuclear Power Station, Unit No. 1
Diablo Canyon Power Plant, Unit Nos. 1 and 2
Edwin I. Hatch Nuclear Plant, Units 1 and 2
Fermi 2
Fort Calhoun Station, Unit 1
Grand Gulf Nuclear Station, Unit 1
H. B. Robinson Steam Electric Plant, Unit No. 2
Hope Creek Generating Station, Unit Nos. 1 and 2
Indian Point Nuclear Generating Unit Nos. 1, 2 and 3
James A. Fitzpatrick Nuclear Power Plant
Joseph M. Farley Nuclear Plant, Units 1 and 2
Millstone Power Station, Unit Nos. 1, 2, and 3
Monticello Nuclear Generating Plant
North Anna Power Station, Unit Nos. 1 and 2
Palisades Nuclear Plant
Palo Verde Nuclear Generating Station, Units 1, 2, and 3
Perry Nuclear Power Plant, Unit No. 1
Pilgrim Nuclear Power Station
Point Beach Nuclear Plant, Units 1 and 2
Prairie Island Nuclear Generating Plant, Units 1 and 2
Salem Nuclear Generating Station, Unit Nos. 1 and 2
San Onofre Nuclear Generating Station, Units 2 and 3
Seabrook Station, Unit No. 1
Sequoyah Nuclear Plant, Units 1 and 2
Shearon Harris Nuclear Power Plant, Unit 1
South Texas Project, Units 1 and 2
Surry Power Station, Unit Nos. 1 and 2
Susquehanna Steam Electric Station, Units 1 and 2
Vermont Yankee Nuclear Power Station
Virgil C. Summer Nuclear Station, Unit No. 1
Vogtle Electric Generating Plant, Units 1 and 2
Waterford Steam Electric Station, Unit 3
Watts Bar Nuclear Plant, Units 1 and 2

Wolf Creek Generating Station

Specifically, the licensees for each of these plants may propose an additional alternate date for full compliance beyond the March 31, 2010 date required by 10 CFR 73.55(a)(1), and if approved by the NRC, would be granted a plant-specific exemption. The proposed action, an exemption extending the schedule for completion of actions required by the revised 10 CFR 73.55, would not involve any physical changes to the reactor(s), fuel, plant structures, support structures, land, or water at each nuclear power plant site. If granted, a plant-specific safety evaluation will be included in the letter to the licensee approving the exemption from the regulation.

The Need for the Proposed Action

The proposed action is needed to continue to exempt up to 40 nuclear power plant sites from being in full compliance with new requirements contained in 10 CFR 73.55. The original implementation date was March 31, 2010 and previous exemptions were granted to several licensees from meeting that date. Another round of exemptions is needed to provide these licensees with additional time to comply with the rule requirements. While licensees completed much of the work required by the 10 CFR 73.55 rule change at their plants by the March 31, 2010 implementation date, affected licensees require additional time to complete all newly required modifications.

Environmental Impacts of the Proposed Action

The NRC regulation, 10 CFR 73.55, requires NRC nuclear power plant licensees to implement various physical protection requirements to prevent radiological sabotage. The NRC has completed its environmental assessment of the proposed action, and has concluded that, should NRC decide to grant a 10 CFR 73.55 compliance date exemption to these plants, extending the implementation deadline would not significantly affect plant safety and would not significantly affect the probability of an accident.

The proposed action would not increase the radiological hazard beyond what was analyzed in the environmental assessment and finding of no significant impact made by the Commission in promulgating its revisions to 10 CFR 73.55 as discussed in the **Federal Register** notice dated March 27, 2009 (74 FR 13926). There would be no change to radioactive effluents and emissions that would increase radiation exposures to plant workers and members of the public. Therefore, no radiological impacts are expected as a result of the proposed action.

In addition, there will be no construction or major renovation of any buildings or structures, nor any ground disturbing activities associated with an extension of the compliance deadline. Licensees would not increase or decrease their workforce, nor is traffic to or around any of the subject power plants expected to increase, as a result of an extension of the compliance deadline. Therefore, providing licensees with additional time to comply with the revised requirements of 10 CFR 73.55 would not alter land use, air quality, and water use (quality and quantity) conditions or National Pollutant Discharge Elimination System permits at each of the nuclear power plants that may be the subject of an exemption request. Aquatic and terrestrial habitat in the vicinity of each power plant; threatened, endangered, and protected species under the Endangered Species Act; and essential fish habitat covered by the Magnuson-Stevens Act would not be affected. In addition, historic and cultural resources, socioeconomic conditions, and minority and low-income populations in the vicinity of each power plant would also not be affected by this action.

As previously noted, in promulgating its amendments to 10 CFR part 73, the Commission prepared an environmental assessment of the rule change and published a finding of no significant impact (10 CFR parts 50, 52, 72, and 73, Power Reactor Security Requirements, 74 FR 13926 (March 27, 2009)). Thus, through the proposed action, the Commission would be granting additional time for the licensees to comply with regulatory requirements for which the Commission has already found no significant impact.

For the foregoing reasons, the NRC concludes that there would be no significant radiological or non-radiological environmental impacts associated with the extension of the implementation date of the new requirements of 10 CFR 73.55.

Environmental Impacts of the Alternatives to the Proposed Action

As an alternative to the proposed action, the NRC considered denial of the proposed actions (*i.e.,* the "no-action" alternative). Denial of the exemption requests would result in no change in current environmental conditions at each of the nuclear power plants.

Denial of the exemption requests would result in the licensees being in non-compliance with 10 CFR 73.55(a)(1) and thus, subject to NRC enforcement action. The end result, however, would still be ultimate licensee compliance with the requirements of 10 CFR 73.55, but with the added expense to both the NRC and the licensees of any enforcement actions. The NRC concludes that the environmental impacts of the proposed exemption and the "no action" alternative are similar.

Alternative Use of Resources

The proposed action does not involve the use of any different resources than those considered in the final environmental statements for the granting of the operating licenses for these nuclear power plants and for those plants that have had their licenses renewed, in NUREG–1437, "Generic Environmental Impact Statement for License Renewal of Nuclear Plants," published in May 1996, and its supplements published from 1999 to the present.

Finding of No Significant Impact

On the basis of the above environmental assessment, which in accordance with 10 CFR 51.32(a)(4), is incorporated into this finding of no significant impact by reference, the NRC concludes that the proposed action constitutes an administrative change (timing) that would not have a significant effect on the quality of the human environment. Accordingly, the NRC has determined not to prepare an environmental impact statement for the proposed action.

Further details with respect to the proposed action will be available as individual licensees request exemptions. Portions of these requests may contain proprietary and safeguards information and, accordingly, will not available to the public. Other parts of these documents may be examined, and/or copied for a fee, at the NRC's Public Document Room (PDR), located at One White Flint North, Public File Area O–1 F21, 11555 Rockville Pike (first floor), Rockville, Maryland 20852. Publicly available records will be accessible electronically from the Agencywide Document Access and Management System (ADAMS) Public Electronic Reading Room on the Internet at the NRC Web site: *http://www.nrc.gov/reading-rm/adams.html.*

Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS should contact the NRC PDR Reference staff by telephone at 1–800–397–4209 or 301–415–4737, or send an e-mail to *pdr.resource@nrc.gov.*

Dated at Rockville, Maryland, this 20th day of December 2010.

For the Nuclear Regulatory Commission.

Robert J. Pascarelli,

Chief, Plant Licensing Branch III–1, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

[FR Doc. 2010–33073 Filed 12–30–10; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[NRC–2010–0265]

Final Regulatory Guide: Issuance, Availability

AGENCY: Nuclear Regulatory Commission.

ACTION: Notice of Issuance and Availability of Regulatory Guide 3.71, Revision 2, "Nuclear Criticality Safety Standards for Fuels and Material Facilities."

FOR FURTHER INFORMATION CONTACT: Tamara D. Powell, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001, telephone: 301–492–3211 or e-mail: *Tamara.Powell@nrc.gov*.

SUPPLEMENTARY INFORMATION:

I. Introduction

The U.S. Nuclear Regulatory Commission (NRC) is issuing a revision to an existing guide in the agency's "Regulatory Guide" series. This series was developed to describe and make available to the public information such as methods that are acceptable to the NRC staff for implementing specific parts of the agency's regulations, techniques that the staff uses in evaluating specific problems or postulated accidents, and data that the staff needs in its review of applications for permits and licenses.

Revision 2 of Regulatory Guide 3.71, entitled, "Nuclear Criticality Safety Standards for Fuels and Material Facilities," was issued with a temporary identification as Draft Regulatory Guide, DG–3030. Regulatory Guide 3.71 provides applicants, licensees, and certificate holders with updated guidance concerning criticality safety standards that the NRC has endorsed for use with nuclear fuels and material facilities. As such, Regulatory Guide 3.71 describes methods that the NRC staff considers acceptable for complying with the NRC's regulations in Title 10, of the Code of Federal Regulations, parts 70, "Domestic Licensing of Special Nuclear Material," and 76, "Certification of Gaseous Diffusion Plants" (10 CFR parts 70 and 76).

The NRC staff has revised Regulatory Guide 3.71 to provide guidance on complying with these portions of the NRC's regulations. This guide describes procedures for preventing nuclear criticality accidents in operations that involve handling, processing, storing, and/or transporting special nuclear material at fuel and material facilities. It also endorses specific nuclear criticality safety standards developed by the American Nuclear Society's Standards Subcommittee 8 (ANS–8), "Operations with Fissionable Materials Outside Reactors." Regulatory Guide 3.71 is not intended for use by nuclear reactor licensees.

II. Further Information

In July 2010, DG–3030 was published with a public comment period of 60 days from the issuance of the guide. The public comment period closed on September 29, 2010. The staff's responses to the public comments received are located in the NRC's Agencywide Documents Access and Management System under Accession Number ML103210349. Electronic copies of Regulatory guide 3.71, Revision 2 are available through the NRC's public Web site under "Regulatory Guides" at *http://www.nrc.gov/reading-rm/doc-collections/.*

In addition, regulatory guides are available for inspection at the NRC's Public Document Room (PDR) located at Room O–1F 21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852–2738. The PDR's mailing address is USNRC PDR, Washington, DC 20555–0001. The PDR can also be reached by telephone: 301–415–4737 or 800–397–4209, by fax: 301–415–3548, and by e-mail: *pdr@nrc.gov*.

Regulatory guides are not copyrighted, and Commission approval is not required to reproduce them.

Dated at Rockville, Maryland this 23rd day of December 2010.

For the Nuclear Regulatory Commission.

John N. Ridgely,

Acting Chief, Regulatory Guide Development Branch, Division of Engineering, Office of Nuclear Regulatory Research.

[FR Doc. 2010–33072 Filed 12–30–10; 8:45 am]

BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[NRC–2009–0444]

Notice of Availability of the Models for Plant-Specific Adoption of Technical Specifications Task Force Traveler TSTF–513, Revision 3, "Revise PWR Operability Requirements and Actions for RCS Leakage Instrumentation"

AGENCY: U.S. Nuclear Regulatory Commission (NRC).

ACTION: Notice of Availability.

SUMMARY: As part of the consolidated line item improvement process (CLIIP), the NRC is announcing the availability of the model application (with model no significant hazards consideration determination) and model safety evaluation (SE) for the plant-specific adoption of Technical Specifications Task Force (TSTF) Traveler TSTF–513, Revision 3, "Revise PWR [pressurized water reactor] Operability Requirements and Actions for RCS [reactor coolant system] Leakage Instrumentation." TSTF–513, Revision 3, is available in the Agencywide Documents Access and Management System (ADAMS) under Accession Number ML102360355. The proposed changes would revise the Standard Technical Specifications (STS) to define a new time limit for restoring inoperable RCS leakage detection instrumentation to operable status and establish alternate methods of monitoring RCS leakage when one or more required monitors are inoperable. Changes to the Technical Specifications (TS) Bases are included, which reflect the proposed changes and more accurately reflect the contents of the facility design bases related to the operability of the RCS leakage detection instrumentation. The CLIIP model SE will facilitate expedited approval of plant-specific adoption of TSTF–513, Revision 3.

Documents: You can access publicly available documents related to this notice using the following methods:

NRC's Public Document Room (PDR): The public may examine and have copied for a fee publicly available documents at the NRC's PDR, Public File Area O1 F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland.

NRC's Agencywide Documents Access and Management System (ADAMS): Publicly available documents created or received at the NRC are available electronically at the NRC's Electronic Reading Room at *http://www.nrc.gov/reading-rm/adams.html.* From this page, the public can gain entry into ADAMS, which provides text and image files of