

participatory activity, it is therefore expected that both the *Evaluation Plan* and *Annual Evaluation Report* would involve and engage across and within an NSF Engine.

Finally, the upper bound estimate for the *Annual Evaluation Report* reflects not only the effort for writing the report but also account for data cleaning, data analysis, and data visualization. We anticipate that the burden for subsequent years to be lower as workflow and cadence will be established after the first year.

Dated: October 22, 2024.

**Suzanne H. Plimpton,**

*Reports Clearance Officer, National Science Foundation.*

[FR Doc. 2024-24913 Filed 10-24-24; 8:45 am]

**BILLING CODE 7555-01-P**

## NUCLEAR REGULATORY COMMISSION

[NRC-2024-0001]

### Sunshine Act Meetings

**TIME AND DATE:** Weeks of October 28, November 4, 11, 18, 25, and December 2, 2024. The schedule for Commission meetings is subject to change on short notice. The NRC Commission Meeting Schedule can be found on the internet at: <https://www.nrc.gov/public-involve/public-meetings/schedule.html>.

**PLACE:** The NRC provides reasonable accommodation to individuals with disabilities where appropriate. If you need a reasonable accommodation to participate in these public meetings or need this meeting notice or the transcript or other information from the public meetings in another format (e.g., braille, large print), please notify Anne Silk, NRC Disability Program Specialist, at 301-287-0745, by videophone at 240-428-3217, or by email at [Anne.Silk@nrc.gov](mailto:Anne.Silk@nrc.gov). Determinations on requests for reasonable accommodation will be made on a case-by-case basis.

**STATUS:** Public.

Members of the public may request to receive the information in these notices electronically. If you would like to be added to the distribution, please contact the Nuclear Regulatory Commission, Office of the Secretary, Washington, DC 20555, at 301-415-1969, or by email at [Betty.Thweatt@nrc.gov](mailto:Betty.Thweatt@nrc.gov) or [Samantha.Miklaszewski@nrc.gov](mailto:Samantha.Miklaszewski@nrc.gov).

**MATTERS TO BE CONSIDERED:**

#### Week of October 28, 2024

*Wednesday, October 30, 2024*

1:00 p.m. Today and Tomorrow Across Region II Business Lines (Public

Meeting) (Contact: Katie McCurry: 404-997-4438)

**Additional Information:** The meeting will be held in the 8th Floor Conference Center, Marquis One Tower, 245 Peachtree Center Avenue NE, Suite 1200, Atlanta, Georgia. The public is invited to attend the Commission's meeting in person or watch live via webcast at the web address—<https://video.nrc.gov/>.

#### Week of November 4, 2024—Tentative

There are no meetings scheduled for the week of November 4, 2024.

#### Week of November 11, 2024

*Thursday, November 14, 2024*

9:00 a.m. Strategic Programmatic Overview of the Operating Reactors and New Reactors Business Lines (Public Meeting) (Contact: Annie Ramirez: 301-415-6780)

**Additional Information:** The meeting will be held in the Commissioners' Hearing Room, 11555 Rockville Pike, Rockville, Maryland. The public is invited to attend the Commission's meeting in person or watch live via webcast at the web address—<https://video.nrc.gov/>.

#### Week of November 18, 2024—Tentative

There are no meetings scheduled for the week of November 18, 2024.

#### Week of November 25, 2024—Tentative

There are no meetings scheduled for the week of November 25, 2024.

#### Week of December 2, 2024

*Thursday, December 5, 2024*

10:00 a.m. Briefing on Equal Employment Opportunity, Affirmative Employment, and Small Business (Public Meeting) (Contact: Erin Deeds: 301-415-2887)

**Additional Information:** The meeting will be held in the Commissioners' Hearing Room, 11555 Rockville Pike, Rockville, Maryland. The public is invited to attend the Commission's meeting in person or watch live via webcast at the web address—<https://video.nrc.gov/>.

#### CONTACT PERSON FOR MORE INFORMATION:

For more information or to verify the status of meetings, contact Wesley Held at 301-287-3591 or via email at [Wesley.Held@nrc.gov](mailto:Wesley.Held@nrc.gov).

The NRC is holding the meetings under the authority of the Government in the Sunshine Act, 5 U.S.C. 552b.

Dated: October 23, 2024.

For the Nuclear Regulatory Commission.

**Wesley W. Held,**

*Policy Coordinator, Office of the Secretary.*

[FR Doc. 2024-24998 Filed 10-23-24; 4:15 pm]

**BILLING CODE 7590-01-P**

## NUCLEAR REGULATORY COMMISSION

[Docket Nos. 72-36, 50-321, and 50-366; NRC-2024-0147]

### Southern Nuclear Operating Company; Edwin I. Hatch Nuclear Plant Units 1 and 2; Independent Spent Fuel Storage Installation; Exemption

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Notice; issuance.

**SUMMARY:** The U.S. Nuclear Regulatory Commission (NRC) issued an exemption to Southern Nuclear Operating Company permitting Edwin I. Hatch Nuclear Plant (Hatch) to maintain three loaded and to load five new 68M multi-purpose canisters with continuous basket shims in HI-STORM 100 Cask System at its Hatch Units 1 and 2 independent spent fuel storage installation in a storage condition where the terms, conditions, and specifications in Certificate of Compliance No. 1014, Amendment No. 11 are not met.

**DATES:** The exemption was issued on October 11, 2024.

**ADDRESSES:** Please refer to Docket ID NRC-2024-0147 when contacting the NRC about the availability of information regarding this document. You may obtain publicly available information related to this document using any of the following methods:

- **Federal Rulemaking Website:** Go to <https://www.regulations.gov> and search for Docket ID NRC-2024-0147. Address questions about Docket IDs in *Regulations.gov* to Stacy Schumann; telephone: 301-415-0624; email: [Stacy.Schumann@nrc.gov](mailto:Stacy.Schumann@nrc.gov). For technical questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- **NRC's Agencywide Documents Access and Management System (ADAMS):** You may obtain publicly available documents online in the ADAMS Public Documents collection at <https://www.nrc.gov/reading-rm/adams.html>. To begin the search, select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1-800-397-4209, at 301-415-4737, or by email to [PDR.Resource@nrc.gov](mailto:PDR.Resource@nrc.gov). The ADAMS accession number for each document

referenced (if it is available in ADAMS) is provided the first time that it is mentioned in this document.

- *NRC's PDR*: The PDR, where you may examine and order copies of publicly available documents, is open by appointment. To make an appointment to visit the PDR, please send an email to [PDR.Resource@nrc.gov](mailto:PDR.Resource@nrc.gov) or call 1-800-397-4209 or 301-415-4737, between 8 a.m. and 4 p.m. eastern time (ET), Monday through Friday, except Federal holidays.

**FOR FURTHER INFORMATION CONTACT:** Heath Stroud, Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001; telephone: 301-287-3664; email: [Heath.Stroud@nrc.gov](mailto:Heath.Stroud@nrc.gov).

**SUPPLEMENTARY INFORMATION:** The text of the exemption is attached.

Dated: October 22, 2024.

For the Nuclear Regulatory Commission.

**Yoira Diaz-Sanabria, Chief,**

*Storage and Transportation Licensing Branch,  
Division of Fuel Management, Office of  
Nuclear Material Safety, and Safeguards.*

#### Attachment—Exemption

#### NUCLEAR REGULATORY COMMISSION

**Docket Nos. 72–36, 50–321, and 50–366**

**Southern Nuclear Operating Company;  
Edwin I. Hatch Nuclear Plant Units 1  
and 2; Independent Spent Fuel Storage  
Installation**

#### I. Background

Southern Nuclear Operating Company (SNC) is the holder of Renewed Facility Operating License Nos. DPR-57 and NPF-5, which authorize operation of the Edwin I. Hatch Nuclear Plant (Hatch), Units 1 and 2 in Baxley, Georgia, pursuant to part 50 of title 10 of the *Code of Federal Regulations* (10 CFR), “Domestic Licensing of Production and Utilization Facilities.” The licenses provide, among other things, that the facility is subject to all rules, regulations, and orders of the U.S. Nuclear Regulatory Commission (NRC) now or hereafter in effect.

Consistent with 10 CFR part 72, subpart K, “General License for Storage of Spent Fuel at Power Reactor Sites,” a general license is issued for the storage of spent fuel in an independent spent fuel storage installation (ISFSI) at power reactor sites to persons authorized to possess or operate nuclear power reactors under 10 CFR part 50. SNC is authorized to operate nuclear power reactors under 10 CFR part 50 and holds a 10 CFR part 72 general license for storage of spent fuel at the Hatch ISFSI. Under the terms of the general license,

SNC stores spent fuel at its Hatch ISFSI using the HI-STORM 100 Cask System in accordance with Certificate of Compliance (CoC) No. 1014, Amendment No. 11.

#### II. Request/Action

By a letter dated June 25, 2024 (Agencywide Documents Access and Management System [ADAMS] Accession No. ML24177A217), as supplemented on July 26, 2024 (ML24208A172), SNC requested an exemption from the requirements of 10 CFR 72.212(a)(2), 72.212(b)(3), 72.212(b)(5)(i), 72.212(b)(11), and 72.214 that require Hatch to comply with the terms, conditions, and specifications of CoC No. 1014, Amendment No. 11 (ML23328A045). If approved, SNC’s exemption request would accordingly allow Hatch to maintain three loaded and to load five multi-purpose canisters (MPCs) with an unapproved, variant basket design with continuous basket shims (CBS) (*i.e.*, MPC-68M-CBS) in the HI-STORM 100 Cask System, and thus, to maintain and load the systems in a storage condition where the terms, conditions, and specifications in CoC No. 1014, Amendment No. 11 are not met.

SNC currently uses the HI-STORM 100 Cask System under CoC No. 1014, Amendment No. 11, for dry storage of spent nuclear fuel in MPC-68M at the Hatch ISFSI. Holtec International (Holtec), the designer and manufacturer of the HI-STORM 100 Cask System, developed a variant of the design with CBS for the MPC-68M, known as MPC-68M-CBS. Holtec performed a non-mechanistic tip-over analysis with favorable results and implemented the CBS variant design under the provisions of 10 CFR 72.48, “Changes, tests, and experiments,” which allows licensees to make changes to cask designs without a CoC amendment under certain conditions (listed in 10 CFR 72.48(c)). After evaluating the specific changes to the cask designs, the NRC determined that Holtec erred when it implemented the CBS variant design under 10 CFR 72.48, as this is not the type of change allowed without a CoC amendment. For this reason, the NRC issued three Severity Level IV violations to Holtec (ML24016A190).

Prior to the issuance of the violation, SNC had loaded three MPC-68M-CBS in the HI-STORM 100 Cask System, which are safely in storage at the Hatch ISFSI. SNC’s loading campaign for the Hatch ISFSI include loading five MPC-68M-CBS in the HI-STORM 100 Cask System beginning in April 2025. While Holtec was required to submit a CoC amendment to the NRC to seek approval

of the CBS variant design, such a process will not be completed in time to inform decisions for this loading campaign. Therefore, SNC submitted this exemption request to allow for the continued storage of the three already loaded MPC-68-CBS, and future loading of five MPC-68M-CBS beginning in April 2025 at the Hatch ISFSI. This exemption is limited to the use of MPC-68M-CBS in the HI-STORM 100 Cask System only for the three already loaded canisters and the planned loading of five specific new canisters using the MPC-68M-CBS variant basket design.

#### III. Discussion

Pursuant to 10 CFR 72.7, “Specific exemptions,” the Commission may, upon application by any interested person or upon its own initiative, grant such exemptions from the requirements of the regulations of 10 CFR part 72 as it determines are authorized by law and will not endanger life or property or the common defense and security and are otherwise in the public interest.

##### A. The Exemption is Authorized by Law

This exemption would allow SNC to maintain three loaded and to load five MPC-68M-CBS in the HI-STORM 100 Cask System at its Hatch ISFSI in a storage condition where the terms, conditions, and specifications in CoC No. 1014, Amendment No. 11, are not met. SNC is requesting an exemption from the provisions in 10 CFR part 72 that require the licensee to comply with the terms, conditions, and specifications of the CoC for the approved cask model it uses. Section 72.7 allows the NRC to grant exemptions from the requirements of 10 CFR part 72. This authority to grant exemptions is consistent with the Atomic Energy Act of 1954, as amended, and is not otherwise inconsistent with NRC’s regulations or other applicable laws. Additionally, no other law prohibits the activities that would be authorized by the exemption. Therefore, the NRC concludes that there is no statutory prohibition on the issuance of the requested exemption, and the NRC is authorized to grant the exemption by law.

##### B. The Exemption Will Not Endanger Life or Property or the Common Defense and Security

This exemption would allow SNC to maintain three loaded and to load five MPC-68M-CBS in the HI-STORM 100 Cask System at the Hatch ISFSI in a storage condition where the terms, conditions, and specifications in CoC No. 1014, Amendment No. 11, are not met. In support of its exemption request,

SNC asserts that issuance of the exemption would not endanger life or property because the administrative controls the applicant has in place prevent a tip-over or handling event, and that the containment boundary would be maintained in such an event. SNC relies, in part, on the approach in the NRC's Safety Determination Memorandum (ML24018A085). The NRC issued this Safety Determination Memorandum to address whether, with respect to the enforcement action against Holtec regarding this violation, there was any need to take an immediate action for the cask systems that were already loaded with non-compliant basket designs. The Safety Determination Memorandum documents a risk-informed approach concluding that, during the design basis event of a non-mechanistic tip-over, the fuel in the basket in the MPC-68M-CBS remains in a subcritical condition.

SNC also provided site-specific technical information, including information explaining why the use of the approach in the NRC's Safety Determination Memorandum is appropriate for determining the safe use of the CBS variant baskets at the Hatch ISFSI. Specifically, SNC described that the analysis of the tip-over design basis event that is relied upon in the NRC's Safety Determination Memorandum, which demonstrates that the MPC confinement barrier is maintained, is documented in the updated final safety analysis report (UFSAR) for the HI-STORM 100 Cask System CoC No. 1014, Amendment No. 11 that is used at the Hatch site. SNC also described its administrative controls for handling of the HI-STORM 100 Cask System at the Hatch ISFSI to prevent a tip-over or handling event. Those controls include operational procedures that demonstrate lift height restrictions and that the system is handled with a single failure proof device, which comply with CoC No. 1014, Amendment No. 11, Appendix A.

Additionally, SNC provided specific information from Hatch's 72.212 Evaluation Report, Revision 27, indicating that during the design basis event of a non-mechanistic tip-over, Hatch's ISFSI would meet the requirements in 10 CFR 72.104, "Criteria for radioactive materials in effluents and direct radiation from an ISFSI or MRS," and 72.106, "Controlled area of an ISFSI or MRS." Specifically, SNC described that, in the highly unlikely event of a tip-over, any potential fuel damage from a non-mechanistic tip-over event would be localized, the confinement barrier would be maintained, and the shielding

material would remain intact. SNC concluded that compliance with 72.104 and 72.106 is not impacted by approving this exemption request.

The NRC staff reviewed the information provided by SNC and concludes that issuance of the exemption would not endanger life or property because the administrative controls that SNC has in place at the Hatch ISFSI sufficiently minimize the possibility of a tip-over or handling event, and that the containment boundary would be maintained in such an event. The staff confirmed that these administrative controls are documented in the technical specifications and UFSAR for the HI-STORM 100 Cask System CoC No. 1014, Amendment No. 11, that is used at the Hatch site. In addition, the staff confirmed that the information provided by SNC regarding Hatch's 72.212 Evaluation Report, Revision 27, demonstrates that the consequences of normal and accident conditions would be within the regulatory limits of the 10 CFR 72.104 and 10 CFR 72.106. The staff also determined that the requested exemption is not related to any aspect of the physical security or defense of the Hatch ISFSI; therefore, granting the exemption would not result in any potential impacts to common defense and security.

For these reasons, the NRC staff has determined that under the requested exemption, the storage system will continue to meet the safety requirements of 10 CFR part 72 and the offsite dose limits of 10 CFR part 20 and, therefore, will not endanger life or property or the common defense and security.

### *C. The Exemption is Otherwise in the Public Interest*

The proposed exemption would allow the three already loaded MPC-68M-CBS in the HI-STORM 100 Cask System to remain in storage at the Hatch ISFSI, and allow SNC to load five MPC-68M-CBS in the HI-STORM 100 Cask System beginning in April 2025 at the Hatch ISFSI, even though the CBS variant basket design is not part of the approved CoC No. 1014, Amendment No. 11. According to SNC, the exemption is in the public interest because being unable to load fuel into dry storage in the future loading campaign would impact SNC's ability to offload fuel from the Hatch reactor units, consequently impacting continued safe reactor operation.

SNC rescheduled its 2024 loading campaign to 2025 and stated that further delaying the future loading campaign would impact its ability to effectively manage the margin to capacity in the

Hatch Units 1 and 2 spent fuel pools. The low spent fuel pool capacity would make it difficult to refuel and present potential risks to fuel handling operations. In addition, a crowded spent fuel pool would challenge the decay heat removal demand of the pool and increase the likelihood of a loss of fuel pool cooling event and a fuel handling accident. Furthermore, Hatch planned the cask loading campaign years in advance based on availability of the specialized resources and equipment. These specialty resources support competing activities and priorities, including spent fuel pool clean-up and refueling outages. Therefore, the available windows to complete cask loading campaigns are limited, and any delays would have a cascading impact on other scheduled specialized activities. SNC also stated that leaving the three loaded CBS canisters is in the public interest because it eliminates the risks associated with dispositioning spent fuel, including increased radiation dose to workers and affecting SNC's ability to manage the spent fuel pool capacity. SNC has ordered an additional five MPC-68M canisters for the planned loading starting in April 2025, that comply with CoC No. 1014, Amendment No. 11. However, due to factors outside of SNC's control, delivery of the compliant canisters is not guaranteed for the April 2025 loading. It would put the 2025 campaign and Hatch's capability for full core offloading of both reactors at risk.

For the reasons described by SNC in the exemption request, the NRC agrees that it is in the public interest to grant the exemption. If the exemption is not granted, in order to comply with the CoC, SNC would have to unload MPC-68M-CBS from the HI-STORM 100 Cask System at the Hatch ISFSI and reload into the older design MPC-68M to restore compliance with the terms, conditions, and specifications of the CoC. This would subject onsite personnel to additional radiation exposures and increase the risk of a possible fuel handling accident. Furthermore, the removed spent fuel would need to be placed in the spent fuel pool until it can be loaded into another storage cask or remain in the spent fuel pool if it is not permitted to be loaded into CBS casks for the future loadings. As described by SNC, this scenario would affect SNC ability to effectively manage the spent pool capacity and reactor fuel offloading at Hatch. In addition, the rescheduling of the specialized resources for the future loading campaign would impact the operations of Hatch.

Therefore, the staff concludes that approving the exemption is in the public interest.

**Environmental Consideration**

The NRC staff also considered whether there would be any significant environmental impacts associated with the exemption. For this proposed action, the NRC staff performed an environmental assessment pursuant to 10 CFR 51.30. The environmental assessment concluded that the proposed action would not significantly impact the quality of the human environment. The NRC staff concluded that the proposed action would not result in any changes in the types or amounts of any radiological or non-radiological effluents that may be released offsite, and there would be no significant increase in occupational or public radiation exposure because of the proposed action. The environmental assessment and the finding of no significant impact was published on October 11, 2024 (89 FR 82645).

**IV. Conclusion**

Based on these considerations, the NRC has determined that, pursuant to 10 CFR 72.7, the exemption is authorized by law, will not endanger life or property or the common defense and security, and is otherwise in the public interest. Therefore, the NRC grants SNC an exemption from the requirements of §§ 72.212(a)(2), 72.212(b)(3), 72.212(b)(5)(i), 72.212(b)(11), and 72.214 with respect to the ongoing storage of three MPC-68M-CBS in the HI-STORM 100 Cask System and a future loading in the HI-STORM 100 Cask System of five new MPC-68M-CBS beginning in April 2025.

This exemption is effective upon issuance.

Dated: October 11, 2024.

For the Nuclear Regulatory Commission.

Yaira Diaz-Sanabria,  
 Chief, Storage and Transportation Licensing Branch, Division of Fuel Management, Office of Nuclear Material Safety, and Safeguards.

[FR Doc. 2024-24917 Filed 10-24-24; 8:45 am]

**BILLING CODE 7590-01-P**

**RAILROAD RETIREMENT BOARD**

**Proposed Collection; Comment Request**

In accordance with the requirement of section 3506 (c)(2)(A) of the Paperwork Reduction Act of 1995 which provides opportunity for public comment on new or revised data collections, the Railroad Retirement Board (RRB) will publish periodic summaries of proposed data collections.

*Comments are invited on:* (a) Whether the proposed information collection is necessary for the proper performance of the functions of the agency, including whether the information has practical utility; (b) the accuracy of the RRB's estimate of the burden of the collection of the information; (c) ways to enhance the quality, utility, and clarity of the information to be collected; and (d) ways to minimize the burden related to the collection of information on respondents, including the use of automated collection techniques or other forms of information technology.

*Title and purpose of information collection:* Railroad Unemployment Insurance Act Applications; OMB 3220-0039.

Under section 2 of the Railroad Unemployment Insurance Act (RUIA) (45 U.S.C. 362), sickness benefits are payable to qualified railroad employees who are unable to work because of illness or injury. In addition, sickness benefits are payable to qualified female employees if they are unable to work, or if working would be injurious, because of pregnancy, miscarriage, or childbirth. Under section 1(k) of the RUIA a statement of sickness, with respect to days of sickness of an employee, is to be filed with the RRB within a 10-day period from the first day claimed as a day of sickness. The Railroad Retirement Board's (RRB) authority for requesting supplemental medical information is section 12(i) and 12(n) of the RUIA. The procedures for claiming sickness benefits and for the RRB to obtain supplemental medical information needed to determine a claimant's eligibility for such benefits are prescribed in 20 CFR part 335.

The forms currently used by the RRB to obtain information needed to determine eligibility for, and the amount of, sickness benefits due a claimant follow: Form SI-1a, Application for Sickness Benefits; Form SI-1b, Statement of Sickness; Form SI-3 (Manual & internet), Claim for Sickness Benefits; Form SI-7, Supplemental Doctor's Statement; Form SI-8, Verification of Medical Information; and Form ID-11A, Requesting Reason for Late Filing of Sickness Benefit. Completion is required to obtain or retain benefits. One response is requested of each respondent. The RRB proposes no changes to Form SI-1a, SI-1b, SI-3, SI-3 (internet), SI-7, SI-8, and ID-11a.

**ESTIMATE OF ANNUAL RESPONDENT BURDEN**

Form No.	Annual responses	Time (minutes)1/	Burden (hours)
SI-1a (Employee) .....	11,179	10	1,863
SI-1b (Doctor) .....	11,179	8	1,490
SI-3 (Manual) .....	100,120	5	8,343
SI-3 (Internet) .....	82,812	5	6,901
SI-7 .....	12,151	8	1,620
SI-8 .....	24	5	2
ID-11A .....	284	4	19
<b>Total .....</b>	<b>217,749</b>	<b>.....</b>	<b>20,238</b>

*Additional Information or Comments:* To request more information or to obtain a copy of the information collection justification, forms, and/or supporting material, contact Kennisha Money at (312) 469-2591 or [Kennisha.Money@rrb.gov](mailto:Kennisha.Money@rrb.gov). Comments

regarding the information collection should be addressed to Brian Foster, Railroad Retirement Board, 844 North Rush Street, Chicago, Illinois 60611-1275 or emailed to [Brian.Foster@rrb.gov](mailto:Brian.Foster@rrb.gov).

Written comments should be received within 60 days of this notice.

**Brian Foster,**  
 Clearance Officer.

[FR Doc. 2024-24818 Filed 10-24-24; 8:45 am]

**BILLING CODE 7905-01-P**